Uncertainty Management for Nuclear Systems Simulation

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Acknowledgement and Research Team

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Introduction

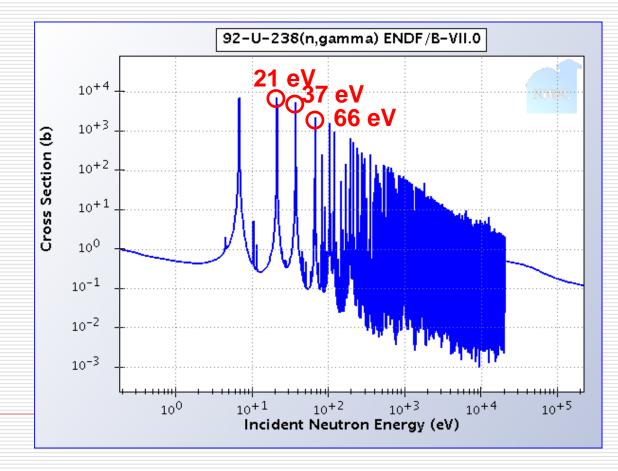
- Recently, modeling and simulation (M&S) recognized as viable tools to achieve optimal design and operation of existing and next generation reactor systems (Gen-IV)
- Design and evaluation strategies projected to reduce reliance on expensive validating experiments and employ accurate M&S as primary design and analysis tool
- M&S must have uncertainty management framework
 - Quantifiable error bounds on simulation results
 - Means to understand various sources of errors
 - Mean to reduce identified sources of errors
 - Means to integrate experiments, and devise their optimal design



Neutron Cross-Section

Many studies proved that nuclear data uncertainties constitute major source of errors in neutronics design calculations

Resonance
Parameter
Uncertainty leads to
0.15% uncertainty in
EOC k-effective
(\$600K in FCC)





Importance of Uncertainty Management

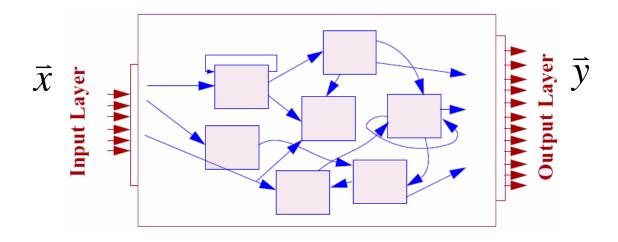
- Define required system design margins
- Identify key input data and associated models contributing most to quantified uncertainties
- Alter design to make it less sensitive to identified key sources of uncertainties
- Optimize experiments design to reduce uncertainties
- Increase design freedom by reducing design margins realized by higher fidelity calculations
- These goals to be achieved via simulation to minimize reliance on expensive experiments

Definitions

Consider a computational model describing an engineering system:

$$\vec{y} = \Theta(\vec{x})$$

- Sensitivity: Rate of Change of output with respect to input
- □ **Uncertainty:** Confidence in calculated results
- Data Assimilation: Reduction of calculations uncertainties



Sensitivity Analysis Goal

□ Given a system model:

$$\vec{y} = \vec{\Theta}(\vec{x})$$

where $\vec{x} \in \square^n$ are input data (physical constants, operating conditions, control parameters, etc.), and $\vec{y} \in \square^m$ are output responses (system attributes of interest to design, operation, and safety)

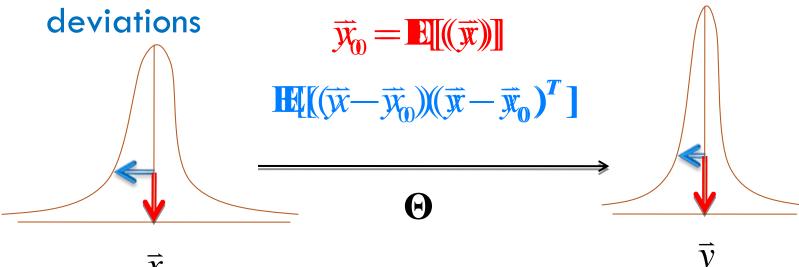
 Calculate <u>at a minimum</u> first order derivatives of output responses with respect to input data

$$\mathbf{\Theta}_{ij} = \frac{\partial y_i}{\partial x_j}, i = 1, ..., m, j = 1, ..., n$$

Uncertainty Analysis Goal

Given system model and input data uncertainties calculate output responses uncertainties.
 Need: Sensitivity Analysis

 Data uncertainties described <u>at a minimum</u> by probability distributions' means and standard



Data Assimilation Goal

- Given measured system responses, adapt model to increase simulation fidelity by accounting for:
 - Modeling errors due to simplifying assumptions (Unclear how to accomplish?)
 - Numerical errors due to discretization
 (Emerging posterior and goal-oriented techniques)
 - **Boundary Conditions** characterizing interaction between various modeling stages: (calls for rigorous approaches)
 - Input data errors

(well-established approaches: requires model inversion, sensitivity analysis, and input data uncertainties)

$$\min_{\mathbf{x}} \left\| \vec{\mathbf{y}}^m - \vec{\Theta}(\vec{\mathbf{x}} + \delta \vec{\mathbf{x}}) \right\|^2 + \alpha^2 \left\| \delta \vec{\mathbf{x}} \right\|^2$$

Uncertainty Management Steps

Linear Approximation

Evaluate sensitivity information

$$\mathbf{\Theta}_{ij} = \frac{\partial y_i}{\partial x_j}, i = 1, ..., m, j = 1, ..., n \Rightarrow \mathbf{\Theta} = \begin{bmatrix} \mathbf{\Theta}_{11} & . & \mathbf{\Theta}_{1n} \\ . & . & . \\ \mathbf{\Theta}_{m1} & . & \mathbf{\Theta}_{mn} \end{bmatrix}$$

Obtain input data covariance matrix

$$\mathbf{C}_{x} = E\left[(\vec{x} - \vec{x}_{0})(\vec{x} - \vec{x}_{0})^{T} \right]$$

Calculate of output data covariance matrix

$$\mathbf{C}_{y} = E\left[(\vec{y} - \vec{y}_{0})(\vec{y} - \vec{y}_{0})^{T}\right] = \mathbf{\Theta}\mathbf{C}_{x}\mathbf{\Theta}^{T}$$

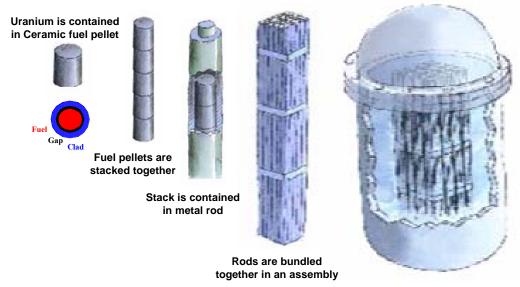
Identify key sources of errors:

$$\left(\mathbf{C}_{x}+\mathbf{\Theta}^{T}\mathbf{\Theta}\right)^{-1}$$

Why UQ Challenging?

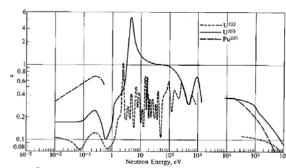
Example: Nuclear Reactors Modeling

- Fully resolved description of reactor is not practical even with anticipated growth in computer power over foreseeable future
- Multi-level homogenization theory adopted to render reactor calculations in practical run times with reasonable accuracy
- Input data: cross-sections, design data, etc.
- Output data: criticality, power, thermal margins, reactivity coefficients, etc.



Assemblies are combined to create the reactor core

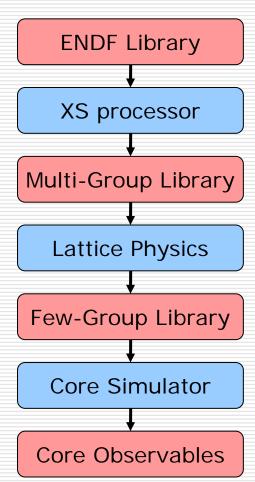
Spatial Heterogeneity of nuclear reactor core



Cross-Sections dependence on neutron energy

BWR Example (Size of I/O streams)

$oxed{\overline{\overline{C}}_{\mathit{ENDF}}}$	10 ⁴ x 10 ⁴	
$ar{f S}_{XP}$	10 ⁷ x 10 ⁴	
$\mathbf{\bar{C}}_{MG} = \mathbf{\bar{S}}_{XP} \mathbf{\bar{C}}_{ENDF} \mathbf{\bar{S}}_{XP}$	10 ⁷ x 10 ⁷	
$ar{f S}_{LP}$	10 ⁶ x 10 ⁷	6.7 hr/ 51 days
$\mathbf{\bar{C}}_{FG} = \mathbf{\bar{S}}_{LP} \mathbf{\bar{C}}_{MG} \mathbf{\bar{S}}_{LP}$	10 ⁶ x 10 ⁶	
$ar{f S}_{CS}$	10 ⁵ x 10 ⁶	5 min
$\mathbf{C}_{CO} = \mathbf{S}_{CS} \mathbf{C}_{FG} \mathbf{S}_{CS}^{TT}$	10 ⁵ x 10 ⁵	





Sensitivity Forward Approach

$$\mathbf{\Theta}\,\delta\,\mathbf{\vec{x}}\,\,\Box\,\,\mathbf{\Theta}(\mathbf{\vec{x}}_0+\delta\,\mathbf{\vec{x}})-\mathbf{\Theta}(\mathbf{\vec{x}}_0)$$

- Perturb input data one-at-a-time to calculate sensitivities of all outputs with respect to the perturbed input
- Suited for problems with few inputs and many outputs
- Variations:
 - Simultaneously perturb all inputs based on their prior PDFs; repeat until the output PDFs converge
 - Suitable for non-Gaussian distributions, and nonlinear systems.

Difficult to infer sensitivity information

$ \frac{\partial y_1}{\partial x_1} $	$\frac{\partial y_1}{\partial x_r}$
$\frac{\partial y_2}{\partial x_1}$	$\frac{\partial y_2}{\partial x_r}$
••	• •
$\frac{\partial y_m}{\partial x_1}$	$\frac{\partial y_m}{\partial x_r}$

Sensitivity Reverse Approach

$$\mathbf{\Theta}^T \delta \, \mathbf{\bar{y}} \, \Box \, ???$$

- Generalized Perturbation Theory
 - Based on select output response, constructs adjoint model to calculate the response sensitivities with respect to all input data
 - Suited for problems with many inputs and few outputs
 - Difficult to implement for legacy codes

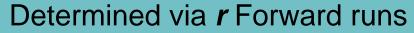
$$\begin{bmatrix} \frac{\partial y_1}{\partial x_1} & \frac{\partial y_1}{\partial x_2} & \dots & \frac{\partial y_1}{\partial x_n} \\ \frac{\partial y_r}{\partial x_1} & \frac{\partial y_r}{\partial x_2} & \dots & \frac{\partial y_r}{\partial x_n} \end{bmatrix}$$



- Replace original I/O streams by mathematical subspaces
- Subspaces are mathematical abstractions denoting change of basis in the I/O streams:
 - Create new I/O variables (called active DOFs).
 - Dimensions of subspaces are much smaller than original I/O streams
 - Each variable (active DOF) is a linear combination of all original variables, with weights reflecting importance of original variables
 - Subspaces identified by means of stochastic approach involving randomized matrix-vector and matrix-transpose-vector products
 - Mathematically, this process is equivalent to finding rank revealing decomposition of sensitivity and uncertainty matrices
 - Requirement: matrices be ill-conditioned



Rank Revealing Decomposition



$$\mathbf{\Theta} \times \delta \, \vec{x} \in \mathrm{span} \left[\vec{u}_1, ..., \vec{u}_{r_r} \right]$$

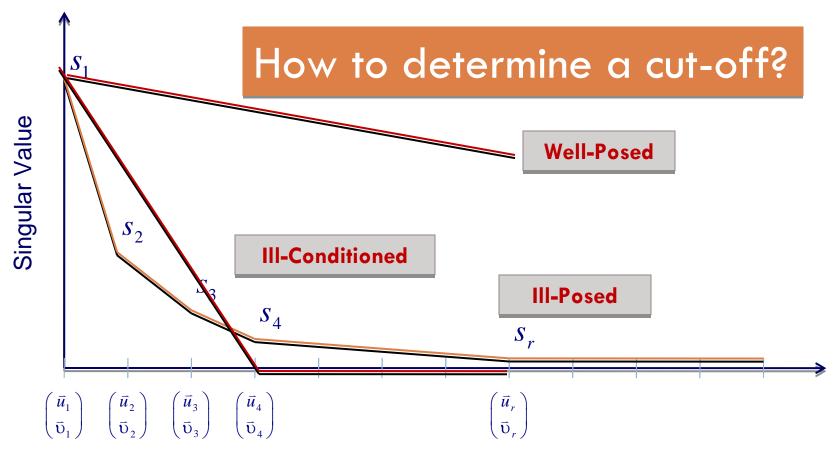
 $\mathbf{\Theta} \times \delta \vec{x} \in \operatorname{span}^{=} \{ \vec{v}_{1}, ..., \vec{v}_{r} \} | \mathbf{\Theta}^{*} \times \delta \vec{v}_{r} \in \operatorname{repan} \{ \vec{v}_{1}, ..., \vec{v}_{r} \}$

Determined via *r* adjoint runs

Active output subspace Active input subspace Active output DOFs Active input DOFs



Singular Values Spectrum



Singular Value Triplet Index

Philosophy of Subspace Methods

- In Euclidean sense, one can change n inputs to a computational model in n different ways, however, for most complex codes, only a subset r<<n leads to noticeable changes in outputs.
- Active Degrees of Freedom denote the various changes in inputs leading to changes in outputs.
- Most outputs of interest to designers and operators are often integral quantities, e.g. power, reactivity, thermal margins, etc., (hence dimensionality reduction)

Active and Inactive DOFs: Example

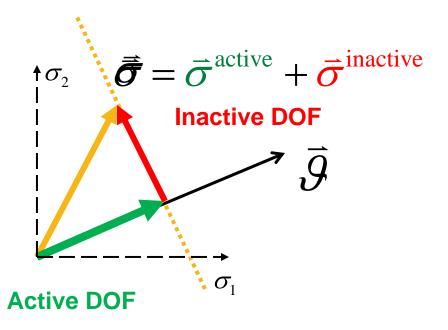
lacktriangleright Consider a simple model with one output response (energy produced from fission) and n input data (fission cross-sections of n different isotopes)

$$E = \sum_{i=1}^{n} \kappa_i N_i \sigma_i \Phi = \vec{\mathcal{G}}^T \vec{\sigma}$$

$$\theta_i = \frac{\delta E}{\delta \sigma_i} = \kappa_i N_i \Phi$$

 \Box Consider **inverse problem**: How to select $\bar{\sigma}$ for some E?

$$\delta \vec{\sigma} \propto \vec{\sigma}^{
m active}$$



Background for Subspace Methods

Dimensionality reduction induced by a multi-level homogenization-type model can be described by Fredholm integral Equation of the first kind

$$\delta y(\boldsymbol{\varpi}) = \int \mathbf{9}(\boldsymbol{\varpi}, t) \, \mathrm{d}x(t) \, dt$$

Every square-integrable *kernel* has mean convergent singular value expansion of the form (Schmidt 1907-1908):

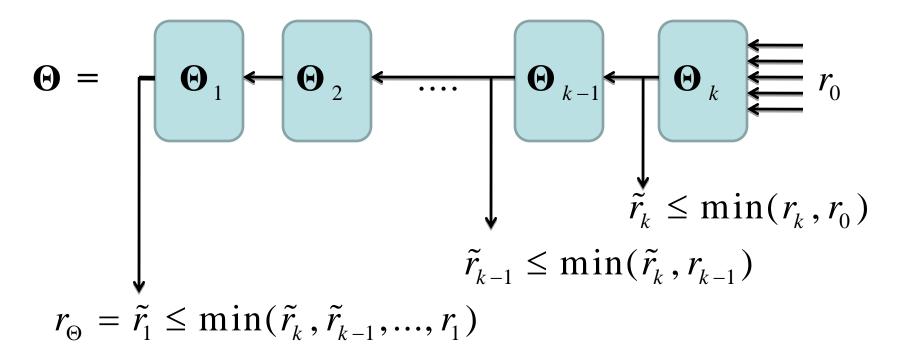
$$\vartheta(\boldsymbol{\varpi},t) = \sum_{i=1}^{\infty} \mu_i u_i(\boldsymbol{\varpi}) v_i(t)$$

Singular Value Decomposition (SVD) is the algebraic version of SVE (Eckart and Young 1936-1939):

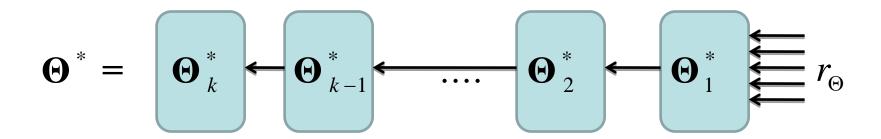
$$\mathbf{\theta}^{m \times n} = \sum_{i=1}^{r} s_i \vec{u}_i \vec{v}_i^T = \mathbf{U}_{m \times r} \mathbf{\Sigma}_{r \times r} \mathbf{V}_{n \times r}^T$$

Consider a multi-level model composed of **k** sub-models (also applies to various components of a single sub-model):

1. Forward Runs:



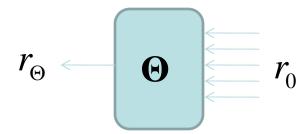
2. Reverse Runs (ex. Adjoint):

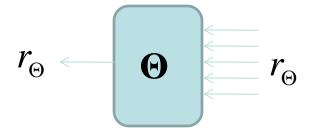


- -Reverse model runs only r_{Θ} times, i.e. rank of overall model.
- -Reverse runs only required for rank-deficient sub-models.

Q: What if reverse model infeasible for a sub-model or component?

A: Given input subspace of dimension r_0 , run forward model r_0 times, and via a RVD, reduce the input subspace to r_{Θ}





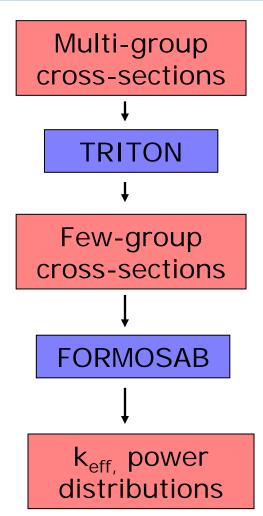
Example 1: Boiling Water Reactor

- Part of GE-Hitachi funded research on 'Development of Adaptive Simulation Algorithms for BWRs'
- Given voluminous amount of data routinely collected from operating nuclear power plants, and maturity of neutronics calculations over past five decades, can one use a data assimilation to enhance agreement between measurements and predictions by adjusting cross-sections?

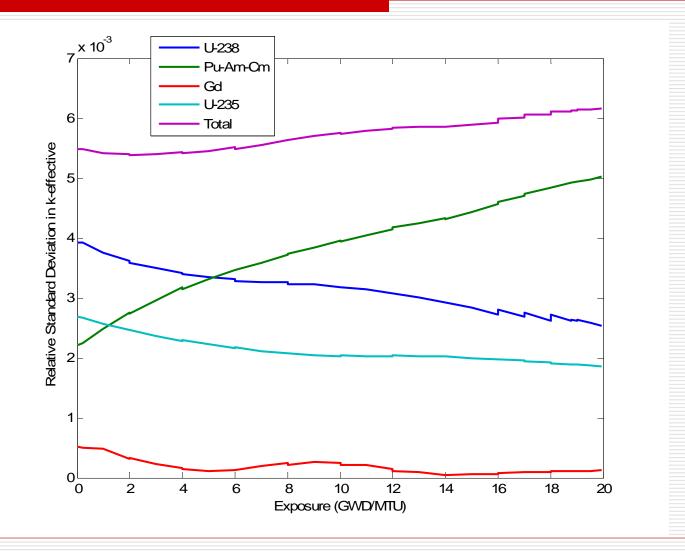


BWR Case Study

- AMPX ORNL ENDF Processing Code System
 - Processes ENDF covariance data into 44 group energy structure
 - SCALE5.0 libraries (PUFF3)
 - 1. 44GROUPV5COV 29 isotopes including H, B, Al, U, Pu, and Minor Actinides et al.
 - 2. 44GROUPANLCOV 30 additional isotopes including Gd, Sm, Zr, et al.
 - SCALE5.1 libraries Evaluations for V5 and
 V6 covariance data
- TRITON ORNL lattice physics code
 - GE14 10x10 lattice design

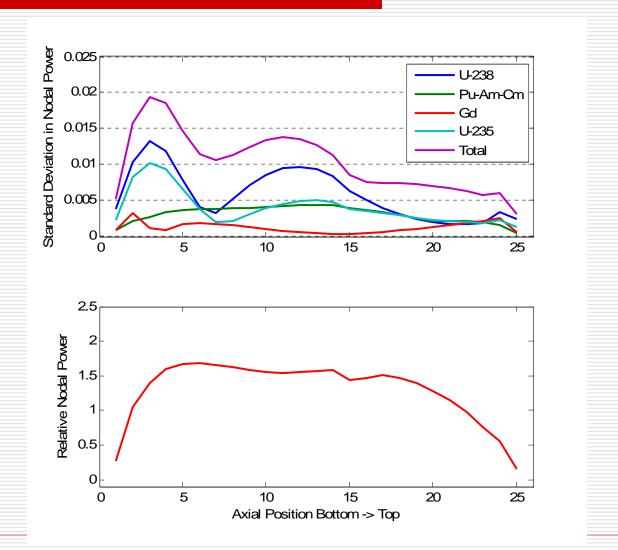


BWR: Few-Group Cross-section Uncertainties



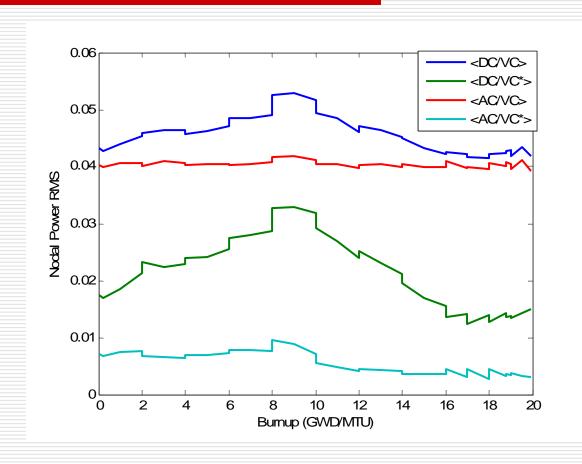


BWR: Power Distribution Uncertainties



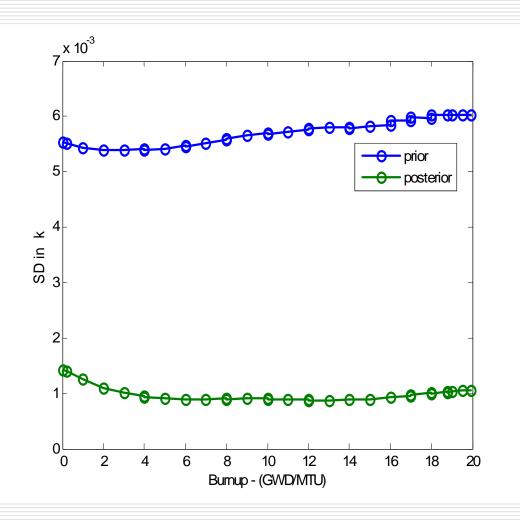


BWR: Data Assimilation "Virtual Approach"



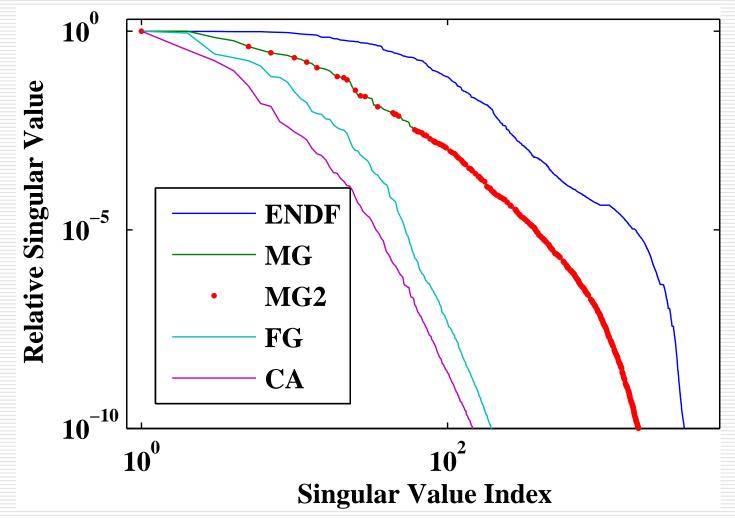


BWR: Data Assimilation "Virtual Approach"





BWR: I/O Streams SVDs



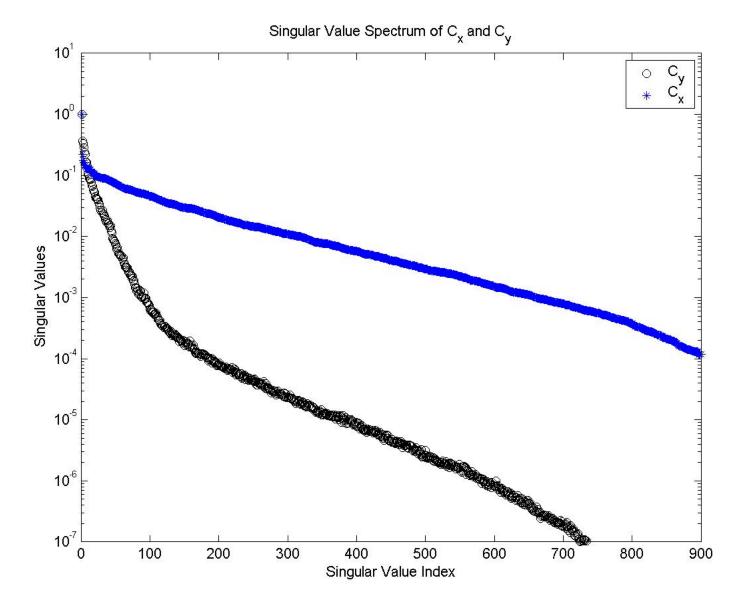


Example 2: Sodium Fast Reactor

- Work part of NERI on 'Management of Data Uncertainties and Optimum Design of Experiments for Gen-IV systems'
- Selected for analysis: ABTR core + ZPR experiments
- Research requires following capabilities:
 - Availability of group x-section uncertainties
 - Propagation of group x-section uncertainties to ABTR key attributes uncertainties
 - Data assimilation for x-sections using ZPR measurements
 - Reevaluation of ABTR's uncertainties using adjusted x-sections

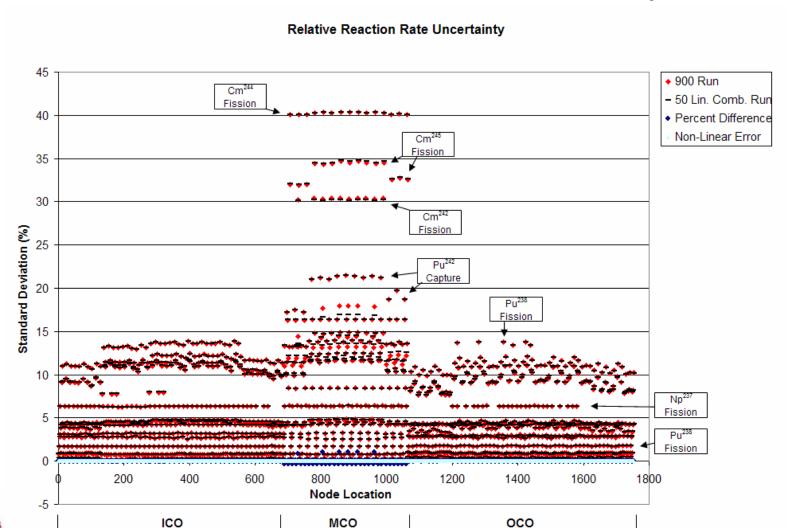


ABTR: I/O Streams SVD





Results: Relative Reaction Rate Uncertainty Data





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Conclusions

- For multi-level models exhibiting reduction in dimensionality through various levels, significant computational savings are possible via a subspace approach
- Only information belonging to 'active' subspaces are communicated between various levels.
- Reverse models only required for the rank-deficient submodels, thus relaxing need for full adjoint capability, which can be quite challenging for linked code system.
- If reverse model infeasible, use a two-step reduction process to identify the active I/O subspace.
- Result is a framework for uncertainty management that can be applied effectively on a routine basis

Future Work

- Extend methodology to adjust resonance parameters directly using a probabilistic Monte Carlo model
- Develop methodology to situations when nonlinear behavior must be considered
 - Weak nonlinearities: Guide deterministic calculations for second order derivatives, i.e. Hessian operators, using active DOFs (r) from linear model (Computational cost ~ r^2)
 - Strong nonlinearities: Hybrid deterministic-probabilistic approach to bias stochastic samples using active DOFs from linearized model
 - Implicit assumption of these developments is that higher order derivatives may be ignored if first order derivatives are small



Questions?

