SECTION V

NMSS HEADQUARTERS FUEL CYCLE SAFEGUARDS INSPECTOR MATERIAL CONTROL AND ACCOUNTING (MC & A) NRC INSPECTOR QUALIFICATION JOURNAL

Applicability

This NRC Inspector Qualification Journal implements NRC Manual Chapter 1246, Appendix A, Section V, by establishing the minimum training requirements for personnel assigned to perform fuel cycle safety inspection (MC & A) activities at fuel facilities.

The NRC Inspector Qualification Journal serves as a guideline for the development of a Qualification Journal, and establishes the minimum training requirements consistent with NRC Manual Chapter 1246. The Qualification Journal must provide traceable documentation to show that minimum requirements are met for each inspector.

The NRC Inspector Qualification Journal consists of a series of qualification guides and signature cards. Each signature card is used to document task completion, as indicated by the appropriate signature blocks. The corresponding qualification guide establishes the minimum knowledge levels or areas of study that must be completed for each signature card.

Most of the qualification guides are divided into sections. The review sections of the qualification guides identify references with general application to the inspector's qualification. The inspector is be expected to have a general familiarity with these references. Other sections of the qualification guides identify specific references that have direct application to an inspection discipline. The inspector is expected to demonstrate detailed knowledge of the inspection discipline specific references.

In order to support the review of upper tier documents, programs, and policies, the inspector's immediate supervisor will assign one or more specific fuel facilities as reference facilities. The selection of a reference facility is intended to provide the inspector's management with the ability to tailor the qualification process to the experience and training level of the inspector, and to meet the inspection needs of the NRC. The use of specific real world material will reinforce the qualification process.

INSPECTOR QUALIFICATION JOURNAL NMSS Headquarters Fuel Cycle Safeguards Inspector - MC & A

Name	Title	Branch	Section	
To complete your qualification as a NMSS Headquarters Fuel Cycle Safeguards Inspector - MC & A you are to complete the following signature cards. All signoffs shall include the signature of the responsible reviewer and the date. Maintain these cards in a notebook along with any background or written material required by the program. This notebook will comprise your NRC Inspector Qualification Journal.				
		Signature When Complete	Date	
1.	NRC Orientation	First Line Supervisor		
2.	Code of Federal Regulations	First Line Supervisor		
3.	Office Instructions /Regional Procedures	First Line Supervisor		
4.	Regulatory Guidance	First Line Supervisor		
5.	NRC Inspection Manual Chapters (MC)	First Line Supervisor		
6.	Industry Codes and Standards	First Line Supervisor		
7.	Inspection Accompaniments	First Line Supervisor		
8.	NRC Management Directives	First Line Supervisor		
9.	Review of Significant Fuel Cycle Events	First Line Supervisor		
10.	Fundamental Nuclear Material Control Plans	First Line Supervisor		
11.	Formal Training	First Line Supervisor		

Qualification Board Requirement Met

Second Level Supervisor or Board Chairman

Recommended as a qualified inspector

Second Level Supervisor

Certification Memo Issued

Second Level Supervisor

Qualification Card 1 NRC Orientation

		<u>Initials</u>	Date
Site Or	ientation		
1.	New employee processing package completed	Employee	
2.	Facility tour and introduction	First Line Supervisor	

NRC Organization Β.

Α.

- Review of NRC headquarters and regional organization 1.
- 2. Discussion of NRC organization

Employee

Qualification Card 2 Code of Federal Regulations (CFR)

	Initials	Date
Familiarization with selected CFR parts completed	Employee	
Discussion completed on CFR parts related to the fuel cycle safeguards (MC & A) inspection program		

First Line Supervisor

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Β.

Qualification Card 3 Office Instructions

Date

A. Familiarization with office policies and procedures

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B. Discussion completed on office policies and procedures

Qualification Card 4 Regulatory Guidance

			Initials	<u>Date</u>
Α.	Review	w of regulatory guidance		
	1.	Regulatory Guides	Employee	
	2.	Information Notices /Bulletins	Employee	
	3.	NUREGs	Employee	
	4.	Generic Letters	Employee	
	5.	Federal Register Notices		
			Employee	
	6.	NRC Branch Technical Positions	Employee	
			Linployee	

B. Discussion of regulatory guidance with application to the fuel cycle safeguards (MC & A) inspection program

Qualification Card 5 NRC Inspection Manual Chapters (MC)

Initials

Date

A. Review of appropriate NRC MCs completed

Employee

B. Discussion of NRC MCs and their relation to the fuel cycle safeguards (MC & A) inspection program

Qualification Card 6 Industry Codes and Standards

	<u>Initials</u>	<u>Date</u>
Review of selected codes and standards completed	Employee	
Discussion of the application of codes and standards in the		

First Line Supervisor

Α.

Β.

fuel cycle safeguards (MC & A) inspection program

Qualification Card 7 Inspection Accompaniments

			Initials	<u>Date</u>
A.	Inspec	tions completed		
	1.	Facility	Employee	
	2	Facility	Employee	
	3	Facility	Employee	
	4.	Facility	Employee	
В.	Discus employ	ssion of inspection and yee's role		
	1.	Facility	First Line Supervisor	
	2.	Facility	First Line Supervisor	
	3.	Facility	First Line Supervisor	
	4.	Facility	First Line Supervisor	

Qualification Card 8 NRC Management Directives

	Initials	<u>Date</u>
Review of selected portions of the NRC Management Directives completed		
	Employee	
Discussion of the application of the NRC Management Directives to the fuel cycle safeguards (MC & A) inspection program		

First Line Supervisor

Α.

Β.

Qualification Card 9 Review of Significant Fuel Cycle Events

		Initials	<u>Date</u>
A.	Review of selected significant historical fuel cycle events	Employee	
В.	Discussion of the importance of these events and lessons learned	First Line Supervisor	

Qualification Card 10 Fundamental Nuclear Material Control Plans

	Initials	Date
Review of selected portions of fundamental nuclear material control plans		
·	Employee	
Discussion of fundamental nuclear		
material control plans and their relationship to the fuel cycle		
safeguards (MC & A) inspection pro- gram		

First Line Supervisor

Α.

Β.

Qualification Card 11 Formal Training

		<u>Initials</u>	Date
Α.	CORE TRAINING:		
1.	Fundamentals of Inspection Course (G-101)	Training Coordinator	
2.	Root Cause/Incident Investigation Workshop (G-205)	Training Coordinator	
3.	Inspecting for Performance Course (G-304)	Training Coordinator	
4.	Effective Communication for NRC Inspectors	Training Coordinator	
5.	OSHA Indoctrination Course (G-111)	Training Coordinator	
6.	Site Access Training (H-100)		
7.	Fundamentals of Nondestructive Assay of Nuclear Material Course (S-602)	Training Coordinator	
8.	Basic Statistics Course or equivalent	Training Coordinator	
9.	Neutron Assay of Nuclear Material Course (S-603)	Training Coordinator	
10.	Gamma-Ray Assay of Nuclear Material Course (S-604)	Training Coordinator	
11.	Integrated Safety Analysis Course (F-103)or Hazards Analysis for DOE SARs and QRAs	Training Coordinator	
12.	Fuel Cycle Processes Directed Self-Study Course (F-201S)	Training Coordinator	

- 13. Uranium Enrichment Process Directed Self-Study Course (F-204S)
- 14. General Health Physics Practices for Fuel Facilities Directed Self-Study Course (F-102S)

Training Coordinator

Training Coordinator

B. SPECIALIZED TRAINING

Other training courses required for inspectors performing inspections in specific areas:

Course Title	Course #	Initials	<u>Initials</u>	<u>Date</u>
		Supervisor	Training Coordinator	
		Supervisor	Training Coordinator	
		Supervisor	Training Coordinator	
		Supervisor	Training Coordinator	

A. Site Orientation

- 1. The qualifying individual should read and complete, as appropriate, the following forms for processing into the NRC:
 - a. Personnel information
 - b. Health insurance elections
 - c. Retirement plan elections
 - d. Savings elections (e.g. U.S. Savings Bonds, TSP, etc.)
 - e. Fitness for Duty requirements and physical examination
 - f. Any other forms which may be required by NRC Office of Human Resources
 - g. Forms for issuance of tagged, controlled NRC equipment
 - h. Payroll forms and time cards
 - i. Regulatory Information Tracking System (RITS)
- 2. The First Line Supervisor should orient the qualifying individual to the facility as follows:
 - a. Tour the facility and introduce the qualifying individual to the staff
 - b. Indicate to the qualifying individual the location of controlled documents, reference material, supplies, office equipment, etc.

B. NRC Organization

- 1. The qualifying individual should review and become familiar with:
 - a. Organizational charts of division, NMSS, regions and headquarters and overall NRC organization (NUREG 0325)
 - b. Role of Headquarters in policy and interpretation of regulations
 - c. Role of NRC General Counsel
 - d. Role of NRC Inspector General
 - e. Role of NRC Public Affairs
 - f. Role of NRC Office of Investigations
 - g. Role of NRC Office of Enforcement

- h. Physical location of NRC offices and regions
- i. Role of NRC as a regulatory agency
 - (1) 10 CFR Part 1 (Organization)
 - (2) Atomic Energy Act of 1954, as amended
 - (3) Energy Reorganization Act of 1974, as amended
 - (4) NRC Enforcement Policy (NUREG 1600)
 - (5) Incident Response Plan (NUREGs 0728 and 0845)
 - (6) Energy Policy Act of 1992
- 2. The First Line Supervisor should discuss NRC organization and role with the qualifying individual to ensure the qualifying individual has a full understanding of NRC's organization and mission and the role of the inspector in that mission.

Qualification Guide 2 Code of Federal Regulations (CFR)

- A. A selection of currently applicable CFR Parts should be made by the First Line Supervisor. The selection should include the references listed below and be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self-study, study-quizzes, briefings, or discussions.
- 1. 10 CFR Part 1 Statement of organization and general information 2. 10 CFR Part 2 Rules of practice for domestic licensing proceedings and issuance of orders Public Records 3. 10 CFR Part 9 4. 10 CFR Part 19 Notices, instructions and reports to workers; inspections Standards for protection against radiation (includes selected 5. 10 CFR Part 20 Questions and Answers, Q & As) 6. 10 CFR Part 21 Reporting of defects and noncompliance 7. 10 CFR Part 30 Rules of general applicability to domestic licensing of byproduct material 8. 10 CFR Part 32 Specific domestic licenses to manufacture or transfer certain items containing byproduct material 9. 10 CFR Part 40 Domestic licensing of source material 10. 10 CFR Part 51 Environmental protection regulations for domestic licensing 11. 10 CFR Part 61 Licensing requirements for land disposal of radioactive waste 12. 10 CFR Part 70 Domestic licensing of special nuclear material 13. 10 CFR Part 71 Packaging and transportation of radioactive material 14. 10 CFR Part 73 Physical protection of plants and materials 15. 10 CFR Part 74 Material control and accounting of special nuclear material 10 CFR Part 75 Safeguards on nuclear material 16. 17. 10 CFR Part 76 Certification of Gaseous Diffusion Plants 10 CFR Part 95 Security facility approval and safeguarding of national security 18. information and restricted data 10 CFR Part 170 19. Fees for facilities, materials, import and export licenses and other regulatory services under the Atomic Energy Act of 1954, as amended

20 10 CFR Part 171 Annual fees for reactor operating licenses, and fuel cycle licenses

and materials licenses, including holders of certificates of compliance, registrations, and quality assurance program approvals and government agencies licensed by NRC

- 21. 29 CFR Part 1910 Occupational Safety and Health Standards
- B. Following completion of the qualifying individual's self study of the listed 10 CFR Parts, a discussion will be held with the qualifying inspector by the First Line Supervisor to test the qualifying inspector's knowledge of these Parts. To the extent possible, recent application of various sections, new regulatory initiatives, and current industry issues should be emphasized.

Qualification Guide 3 Office Instructions/Regional Procedures

- A. Office/Division Policies and Procedures
 - 1. Read the NMSS Policy and Procedures Manual
 - 2. The qualifying individual should review the NMSS policies and practices on:
 - a. Travel, including Management Directive 14.1 Official Temporary Duty Travel
 - b. Telephone use
 - c. Policies on use of annual leave and sick leave and excused leave, including Bulletin 4135, Leave Administration
 - d. Work schedule, including NRC Appendix 4136, Hours of Work and Premium Pay
 - e. Use of government equipment, including computers (NUDOCS and ADAMS) and Management Directive 13.1, Property Management
 - f. Union activities, including Management Directive 10.102, Labor-Management Relations Program for Federal Employees
 - g. Communications outside NRC
 - h. Policies on outside employment and acceptance of gifts
 - i. Participation in political activities
 - j. Routing of mail and procedures for sending mail and materials (via U.S. Mail, Federal Express, etc.), including Management Directive 3.23, Mail Management
 - k. Ordering of documents (e.g NUREGs)
 - I. Emergency and evacuation procedures
 - m. Employee appraisal system and Individual Development Plan (IDP)
 - (1) Employee trial period (Management Directive 10.14 Employment and Staffing)
 - (2) Employee appraisals (Management Directive 10.67, Non-SES Performance Appraisal System)
 - n. Differing Professional Views or Opinions (Management Directive 10.159, General Personnel Management Provisions)
- B. The First Line Supervisor should discuss these policies and practices with the qualifying individual to ensure that the qualifying individual has a full and complete understanding.

- A. A selection of currently applicable regulatory guidance should be identified by the First Line Supervisor. It should be noted that not all of the referenced regulatory guides will be applicable to each inspector's area of responsibility. These references should be selected from those listed below and should be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. The review may be accomplished by selfstudy, study-quizzes, briefings, or discussions. Note that many Regulatory Guides reference or endorse industry codes and standards listed in Qualification Guide 6. Study of corresponding and subtier codes and standards is recommended.
 - 1. Regulatory Guides (use latest revision)
 - 3.71 Nuclear Criticality Safety Standards for Fuels and Materials Facilities (Draft DG-3013 published 1/98) (Guide Withdraws RG 3.1, 3.4, 3.43, 3.45, 3.47, 3.57, 3.58, 3.68, 3.70, and 8.12)
 - 5.7 Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas
 - 5.10 Selection and Use of Pressure Sensitive Seals on Containers for Onsite Storage of Special Nuclear Material
 - 5.11 Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste
 - 5.12 General Use of Locks in the Protection and Control of Facilities and Special Nuclear Materials
 - 5.13 Conduct of Nuclear Material Physical Inventories
 - 5.15 Tamper-Indicating Seals for the Protection and Control of Special Nuclear Material
 - 5.21 Nondestructive Uranium 235 Enrichment Assay by Gamma Spectrometry
 - 5.26 Selection of Material Balance Areas and Item Control Areas
 - 5.31 Specially Designed Vehicle with Armed Guards for Road Shipment of Special Nuclear Material
 - 5.37 In Situ Assay of Enriched Uranium Residual Holdup
 - 5.67 Material Control and Accounting For Uranium Enrichment Facilities Authorized to Produce Special Nuclear Material of Low Strategic Significance
 - 8.1 Radiation Symbol
 - 8.5 Criticality and Other Interior Evacuation Signals
 - 8.7 Instructions For Recording and Reporting Occupational Radiation Exposure Data
 - 8.10 Operating Philosophy for Maintaining Occupational Radiation Exposure As

Low As Is Reasonably Achievable

	8.13 Ins	struction Concerning Prenatal Radiation Exposure	
2.	Information Notices(IN) and Bulletins (BL)		
	IN 89-24	Nuclear Criticality Safety	
	IN 90-09	Extended Interim Storage of Low-Level Radioactive Waste by Fuel Cycle and Materials Licensees	
	IN 91-84	Problems with Criticality Alarm Components/Systems	
	IN 92-11	Soil and Water Contamination at Fuel Cycle Facilities	
	IN 92-14	Uranium Oxide Fires at Fuel Cycle Facilities	
	IN 93-60, Supplement 1	Reporting Fuel Cycle and Materials Events to the NRC Operations Center	
	IN 94-73	Clarification of Criticality Reporting Criteria	
	IN 98-05	Criminal History Record Information	
	IN 98-35	Threat Assessments and Considerations of Heightened Physical Protection Measures	
	IN 99-16	Federal Bureau of Investigation's Nuclear Site Security Program	
	BL 91-01 Supplement 1	Reporting Loss of Criticality Safety Controls	
	Others as selected by the First Line Supervisor		
3.	NUREGs (latest revision, where applicable)		
	NUREG 1065	Acceptable Standard Format and Content for the Fundamental Nuclear Material Control Plan Required for LEU Facilities	
	NUREG 1189, Vol. 1 and 2	Assessment of the Public Health Impact From the Accidental Release of UF_6 at the Sequoyah Fuels Corporation Facility at Gore, Oklahoma	
	NUREG 1198	Release of UF $_{\rm 6}$ From A Ruptured Model 48Y Cylinder at Sequoyah Fuels Corporation Facility	
	NUREG 1198 Supplement No.	Release of UF ₆ From a Ruptured Model 48Y Cylinder at 1 Sequoyah Fuels Corporation Facility: Lessons-Learned Report	
	NUREG 1280	Standard Format and Content Acceptance Criteria for the MC & A Reform Amendment	
	NUREG 1324	Proposed Method for Regulating Major Materials Licensees	
	NUREG 1450	Potential Criticality Accident at the General Electric Nuclear Fuel and Component Manufacturing Facility, May 29, 1991	

NUREG 1520	Standard Review Plan for the Review of a License Application for a Fuel Cycle Facility
NUREG 1600	General Statement of Policy and Procedures for NRC Enforcement Actions
NUREG/BR 0006	Instructions for Completing Nuclear Material Transaction Reports
NUREG/BR 0007	Instructions for Completing Material Balance Reports and Physical Inventory Listings
NUREG/BR 0096	Instructions and Guidance for Completing Physical Inventory Summary Reports
NUREG/CR 2078	Handbook of Nuclear Safeguards Measurement Methods
NUREG/CR 4604	Statistical Methods For Nuclear Material Management
NUREG/CR 5734	Recommendations to the NRC on Acceptable Standard Format and Content for the Fundamental Nuclear Material Control Plan Required for LEU Enrichment Facilities

Others as selected by the First Line Supervisor

- 4. Generic Letters(GL)
 - GL 91-003 Reporting of Safeguard Events
 - GL 95-001 NRC Staff Technical Position on Fire Protection For Fuel Cycle Facilities

Others as selected by the First Line Supervisor

5. Federal Register Notices

U.S. Nuclear Regulatory Commission, "Guidance on Management Controls/Quality Assurance, Requirements for Operation, Chemical Safety, and Fire Protection for Fuel Cycle Facilities," *Federal Register* 54 (No. 53), 11590-11598, March 21, 1989

Others as selected by the First Line Supervisor

B. The application of these guidance documents to the fuel cycle safeguards (MC & A) inspection program should be studied in detail by the qualifying individual and covered by the First Line Supervisor in discussions, interviews, or oral quizzes.

Qualification Guide 5 NRC Inspection Manual Chapters (MC)

- A selection of currently applicable NRC MC and Inspection Procedure (IP) references with direct application to the fuel cycle safeguards (MC & A) inspection program should be identified by the First Line Supervisor. The application of the specific references to the fuel cycle safeguards (MC & A) inspection program should be studied in detail by the qualifying individual.
- 1. REPORTS/COMMUNICATIONS/FOLLOW-UP
 - MC 0030 Policy and Guidance for Development of NRC Inspection Manual Programs
 - MC 0230 Morning Report
 - MC 0610 Inspection Reports
 - MC 0620 Inspection Documents and Records
 - MC 0720 NRC Bulletins and Information Notices
 - MC 0730 Generic Communications Regarding Materials and Fuel Cycle Issues
 - MC 0801 Inspector Feedback
 - MC 1120 Preliminary Notifications
 - IP 92701 Followup
 - IP 92703 Followup of Confirmatory Action Letters
- 2. INSPECTIONS
 - MC 0300 Announced and Unannounced Inspections
 - MC 0312 Technical Assistance for Radiation Safety Inspections at Nuclear Fuel Cycle Facilities and Material Licensee's Sites
 - MC 0630 Analysis of the Impact of Noncompliance with Physical Security Requirements) MC 1246 Formal Qualification Programs in Nuclear Material Safety and Safeguards
 - Program Area MC 2600 Eucl Cycle Eacility Operational Safety and Safeguarda
 - MC 2600 Fuel Cycle Facility Operational Safety and Safeguards Inspection Program
 - MC 2603 Inspection of the Nuclear Chemical Process Safety Program at Fuel Cycle Facilities
 - MC 2604 Licensee Performance Review
 - MC 2681 Safeguards Inspection of Fuel Facilities, Transport of SNM and Irradiated Fuel, and SNM Imports and Exports
 - MC 2682 Technical Assistance for Safeguards MC & A Inspections at Fuel Facilities
 - MC 2800 Materials Inspection Program
 - MC 8800 Fuel Facility Inspection
 - IP 88102 Surveillance Observations
- 3. INTERACTIONS WITH OTHER FEDERAL AGENCIES
 - MC 1007 Interfacing Activities Between Regional Offices of NRC and OSHA
 - IP 93001 OSHA Interface Activities
- 4. INCIDENT RESPONSE
 - MC 1300Incident Response Actions Responsibility and AuthorityMC 1301Response to Radioactive Material Incidents that Do Not Require Activation of the
NRC Incident Response Plan
 - MC 1302 Action Levels for Radiation Exposures and Contamination Associated with

- Materials Events Involving Members of the Public
- MC 1360 Use of Physician and Scientific Consultants in the Medical Consultant Program
- IP 88003 Reactive Inspection for Events at Fuel Cycle Facility Program
- 5. WASTE MANAGEMENT

MC 8400 Radioactive Waste Management

6. FUEL CYCLE SAFETY PROGRAM

MC 8100	Physical Security
MC 8500	Material Control and Accountability
MC 8800	Fuel Facility Inspection

- 7. RADIATION PROTECTION
 - MC 8300 Radiation Protection
- 8. OTHER

MC 1100	Notification of Significant Meetings
MC 1201	Conduct of Employees
MC 2900	Performance Appraisal Program

B. The First Line Supervisor will hold discussions, interviews, or oral quizzes to test the qualifying individual's knowledge and understanding of the application of the selected references to the fuel cycle safeguards (MC & A) inspection program.

Qualification Guide 6 Industry Codes and Standards

- A. A selection of currently applicable industry codes and standards should be identified by the First Line Supervisor. These references should include those listed below and be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self study, study quizzes, briefings, or discussions.
 - 1. American National Standards Institute (ANSI)

ANSI N15.15	Nuclear Materials - Assessment of the Assumptions of Normality (Employing Individual Observed Values)
ANSI N15.18	Nuclear Materials - Mass Calibration Techniques for Control
ANSI N15.19	Nuclear Materials Control - Volume Calibration Techniques
ANSI N15.20	Guide to Calibrating Nondestructive Assay Systems
ANSI N15.22	Nuclear Materials - Plutonium-Bearing Solids - Calibration Techniques for Calorimetric Assay
ANSI N15.28	Nuclear Materials Control - Guide for Qualification and Certification of Safeguards and Security Personnel
ANSI N15.37	Guide to the Automation of Nondestructive Assay Systems for Nuclear Material Control
ANSI N15.41	Derivation of Measurement Control Programs - General Principles
ANSI N15.51	Nuclear Materials Management - Measurement Control Program - Nuclear Materials Analytical Chemistry Laboratory
ANSIN15.54	Instrumentation - Radiometric Calorimeters - Measurement Control Program
ANSI/ANS 8.1	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors
ANSI/ANS 8.3	Criticality Accident Alarm System
ANSI/ANS 8.5	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of Fissile Material
ANSI/ANS 8.7	Guide for Nuclear Criticality Safety in the Storage of Fissile Materials
ANSI/ANS 8.9	Nuclear Criticality Safety Criteria for Steel-Pipe Intersections Containing Aqueous Solutions of Fissile Material
ANSI/ANS 8.17	Criticality Safety Criteria for the Handling, Storage, and
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Transportation of LWR Fuel Outside Reactors

ANSI/ANS 8.19 Administrative Practices for Nuclear Criticality Safety

ANSI/ANS 8.20 Nuclear Criticality Safety Training

Others as selected and documented by the First Line Supervisor

ANSI NFPA Standards as selected and documented by the First Line Supervisor (NOTE: a list is provided in Section 8.4.2 of NUREG 1520).

-Institute of Electrical and Electronic Engineers IEEE standards associated with security and analysis equipment

2. NRC Accepted HP Computer Codes

PC-DOSE Varskin RASCAL REMIT

3. International Atomic Energy Agency (IAEA)

Safety Series No. 1, 25, 33, 38

Technical Report Series No. 120 and 133

4. Other

DP-1014, Maximum Safe Limits for Slightly Enriched Uranium and Uranium Oxide, H. K. Clark, Du Pont de Nemours and Co., Aiken, SC, 1966

Draft Regulatory Guide, DOE/NCT-04, A Review of Criticality Accidents, W. R. Stratton, Revised by D. R. Smith, U.S. DOE, March 1989

B. The First Line Supervisor should test the qualifying individual's knowledge of application of these codes and standards to the fuel cycle safeguards (MC & A) inspection program by discussions, interviews, or oral quizzes.

Qualification Guide 7 Inspection Accompaniments

- A. Each inspector should accompany certified inspectors on at least four inspections. At least two of these inspections should be performed at a facility other than the designated lead plant.
- B. The following is a guide for material that should be studied and discussed with the inspector in charge during these inspection accompaniments. The First Line Supervisor will discuss these items, as appropriate, following each inspection accompaniment.
 - 1. The Inspection Program

MC 2600 Fuel Facility Inspection Program

White Paper Risk-Informed and Performance-Based Regulation

2. Scheduling and Preparation for Inspections

MC 0300 Announced and Unannounced Inspections

- 3. Scope of Inspection
- 4. Entrance/Exit Interviews
- 5. Conduct of Inspection, Accumulation of Data
- 6. Post-inspection Activities of Inspectors

MC 0610 Inspection Reports

MC 0630 Analysis of the Impact of Noncompliance with Physical Security Requirements

MC 1100 Notification of Significant Meetings

7. Morning Reports

MC 0230 Morning Report

8. Non-routine Licensee Events

MC 1110 Potential Abnormal Occurrences

MC 0325 Augmented Inspection Team

Management Directive 8.3 NRC Incident Investigation Program

Management Directive 8.9 Accident Investigation

- Preliminary Notification
 MC 1120 Preliminary Notifications
- Bulletins/Information NoticesMC 0720 NRC Bulletins and Information Notices

11. Use of Consultants of NRC

MC 1360 Use of Physician and Scientific Consultants in the Medical Consultant Program

Management Directive 10.6 Use of Consultants & Experts

12. Allegations and Investigations

Management Directive 8.8 Management of Allegations

13. Communication outside NRC

Management Directive 5.5 Public Affairs Program

Management Directive 3.6 Distribution of Unclassified NRC Staff/Contractor-Generated Reports

Qualification Guide 8 NRC Management Directives

- A. A selection of currently applicable NRC Management Directive (MD) references should be identified by the First Line Supervisor. These references should include those listed below and be documented. The qualifying inspector should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self-study, study-quizzes, briefings, or discussions. The selection should include:
 - 1. NRC MD 9.1 Organization Management 2. NRC 9.29 Organization and Function of Regional Offices 3. **NUREG 0325 USNRC** Functional Organization Chart 4. **NRC MD 3.2** Privacy Act Freedom of Information Act 5. NRC MD 3.1 6. NRC MD 10.130 Safety and Health Program Under the Occupational Safety and Health Act 7. NRC MD 10.131 Protection of NRC Employees Against Ionizing Radiation 8. NRC MD 14.1 Official Temporary Duty Travel 9. NRC MD 10.159 **Differing Professional Views or Opinions** 10. NRC MD 10.42 Hours of Work and Premium Pay 11. NRC MD 10.43 Time and Attendance Reporting 12. Non-SES Performance Appraisal System NRC MD 10.67 13. NRC MD 10.101 **Employee Grievances NRC MD 8.3** 14. NRC Incident Investigation Program 15. NRC MD 8.8 Management of Allegations

B. Application of the selected NRC Management Directives to the fuel cycle safeguards (MC & A) inspection program will be discussed with the qualifying individual by the First Line Supervisor to test the qualifying individual's knowledge.

Qualification Guide 9 Review of Significant Fuel Cycle Events

- A. A selection of significant historical fuel cycle related events should be identified by the First Line Supervisor. These events should be studied in detail by the qualifying individual. Such events would include the following. Other events may be chosen but in any case the events chosen should be documented.
 - 1. Sequoyah Fuels accidents in 1986 and in 1992
 - 2. Potential criticality at the GE Wilmington plant in 1991
 - 3. Y-12 criticality accident in 1958
 - 4. UO_2 fires at fuel fabrication plants
 - 5. United Nuclear Wood River Junction in 1964
- B. The First Line Supervisor should discuss the selected events in detail with the qualifying inspector and go over recommendations made, lessons learned, and changes identified to prevent recurrence. The relevance of the event to the overall fuel cycle safeguards (MC & A) inspection program should be stressed.

Qualification Guide 10 Fundamental Nuclear Material Control Plans

- A. The inspector should become generally familiar with several fundamental nuclear material control plans and/or the license applications. The inspector's review should include the safety analysis and security plan associated with the license application with special emphasis in the following areas, as appropriate.
 - 1. Site characteristics
 - 2. Process systems
 - 3, Safety features
 - 4. Measurement systems
 - 5. Material control systems
- B. Each section of a reference facility's fundamental nuclear material control plan and/or license application should be reviewed with an emphasis on its application to the fuel cycle safeguards MC & A inspection program. After reviewing the fundamental nuclear material control plan and/or license application, the inspector will be able to specifically address its application to the MC & A inspection program. The inspector may demonstrate knowledge through discussions, interviews or quizzes. These discussion activities should be conducted by the First Line Supervisor or alternatively, by senior inspectors to illustrate recent application of regulatory guidance to the inspection program. Completion of the discussion activities must be documented.

Qualification Guide 11 Formal Training

The standards for each Training Course are provided in the NRC Technical Training Division Course Catalog and will not be duplicated in the Qualification Guide.