

Modeling of NSTX Plasmas with the Tokamak Simulation Code (TSC)

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PPPL

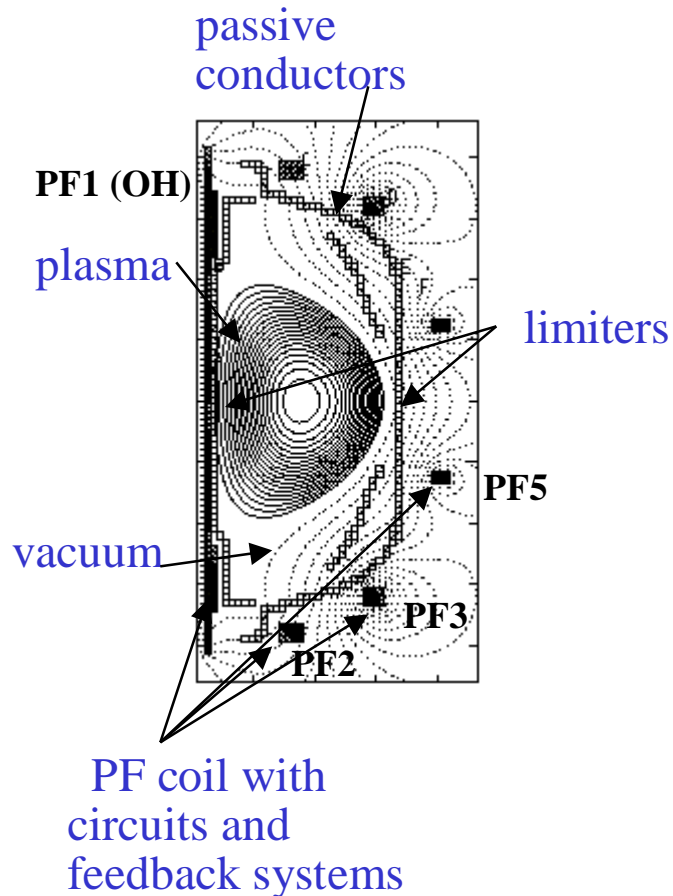
NSTX Research Forum

PPPL Nov 29, 2001

Summary and Overview

- The Tokamak Simulation Code (TSC) is widely used for the design of new experiments...in particular to predict flux requirements
- TSC can read a NSTX data file with coil currents, line averaged density, and magnetics measurements (Thanks to J. Menard) .
- Good agreement with magnetics data can be obtained, but this normally requires adjusting χ_e and/or χ_I multiplier in time.
- TSC has been coupled with a ballooning stability code and with DCON and PEST to provide stability predictions for NSTX operation
- TSC can also model CHI experiments where force-free current exits and enters vessel due to applied poloidal voltage..assumes 2D
- Some upgrades are in the works that should make TSC more useful as a tool for developing integrated scenarios

Tokamak Simulation Code (TSC)



- TSC models the evolution of a **free-boundary** axisymmetric toroidal plasma on resistive and energy confinement time scales.
- The **surface-averaged transport** equations for the pressures and densities are solved in magnetic flux coordinates using matrix implicit method
- An **arbitrary transport model** can be used,
- **Neoclassical-resistivity, bootstrap-current, auxiliary-heating, current-drive, alpha-heating, radiation, pellet-injection, sawtooth, and ballooning-mode transport models** are all available.
- As an option, **circuit equations** are solved for all the poloidal field coil systems with the effects of induced currents in passive conductors included.
- **Realistic feedback systems** can be defined to control the time evolution of the plasma current, position, and shape.
- **Open field lines** can be included, and the halo current is computed as part of the calculation

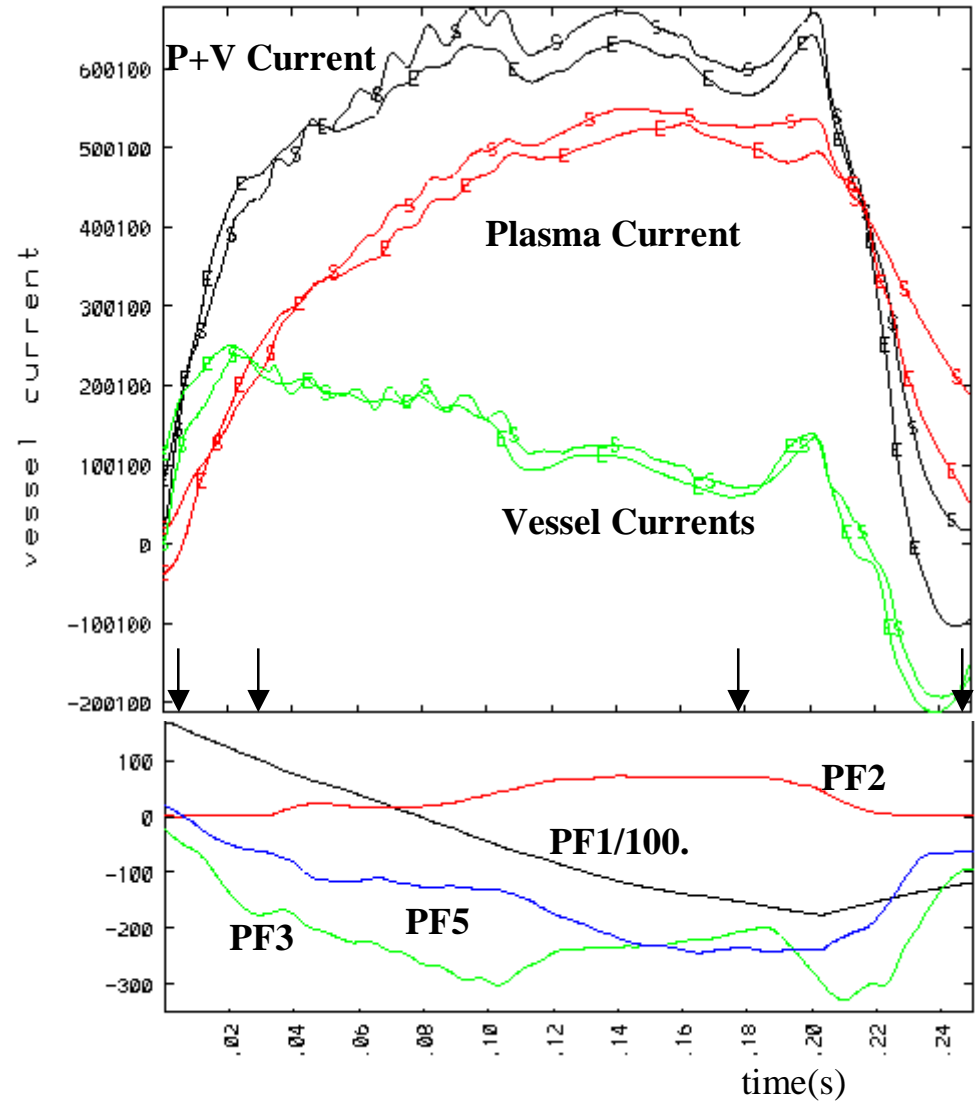
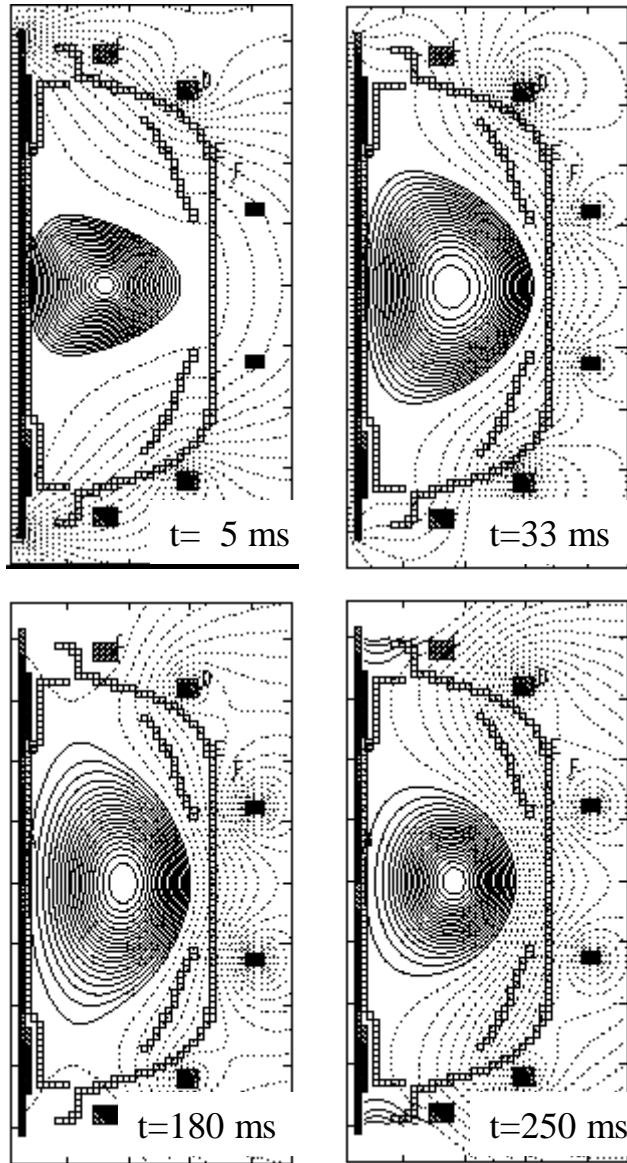
Comparison of TSC with Predictive Transp

Capability	PTRANSP	TSC
Te/Ti/V ϕ , q prediction	yes	yes
Density prediction & pellets	no +2	yes
NBI heating and CD	yes	simple model*
ICRF high harmonic	no +3	no +3
Lower Hybrid	yes	yes
Impurity radiation	no +1	yes
GLF23 transport model	no +2	yes
Fixed boundary equilibrium	yes	no
Free boundary equilibrium	no	yes

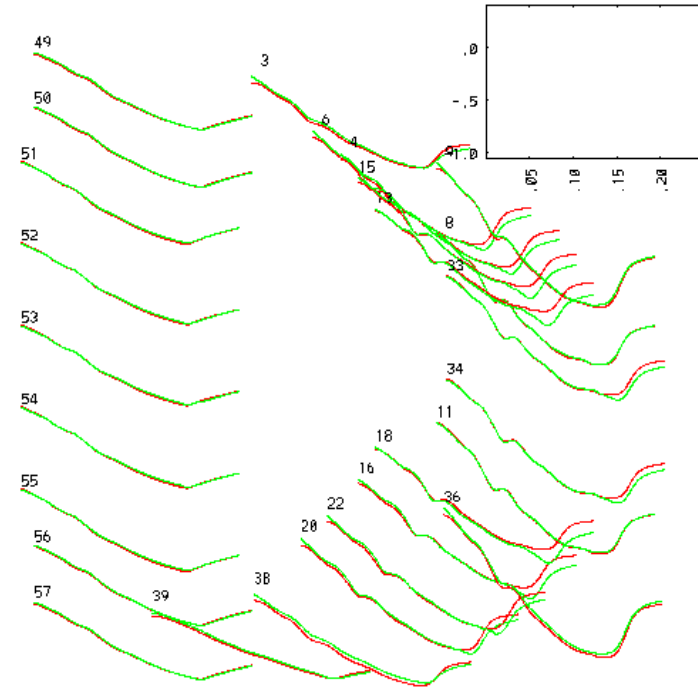
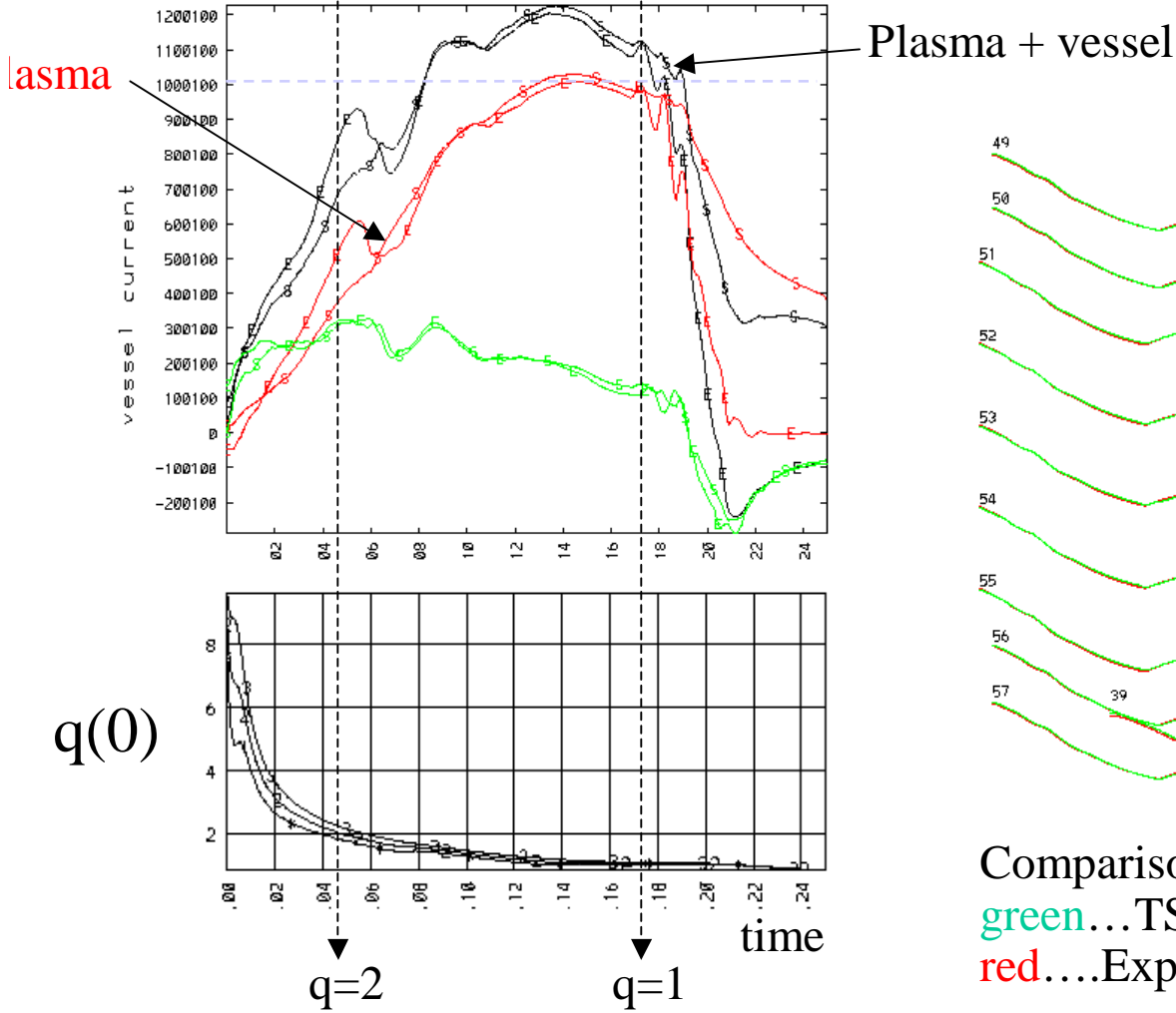
* Will upgrade to TRANSP beam package when it is available

NSTX shot 100920

Predict plasma current using actual coil currents and standard transport model

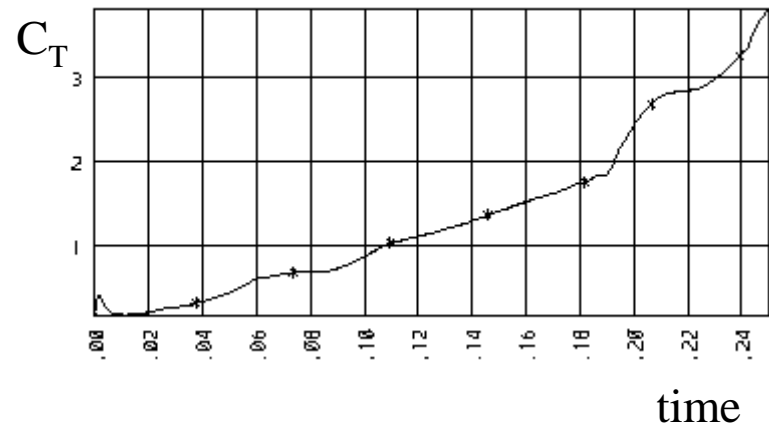
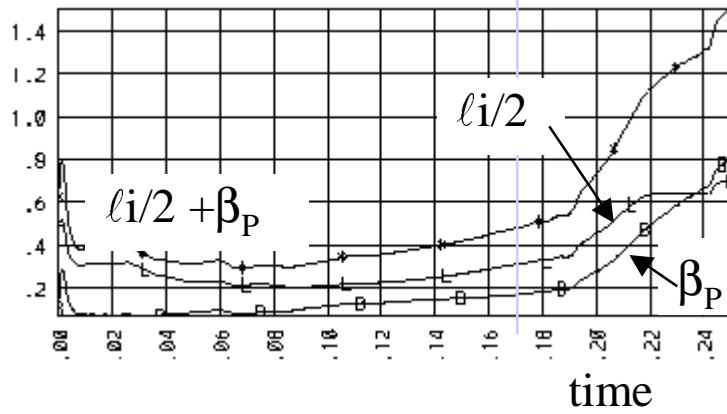
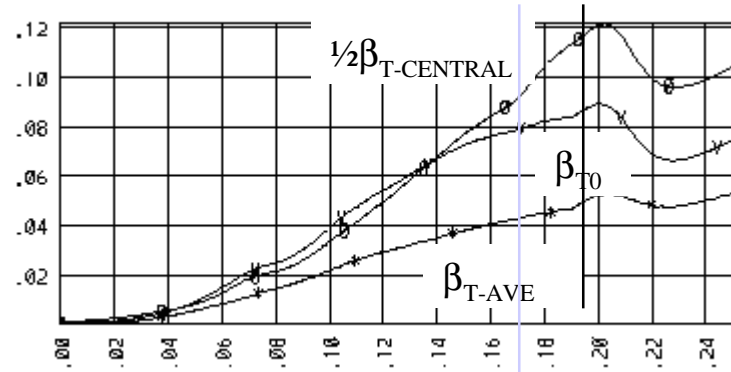
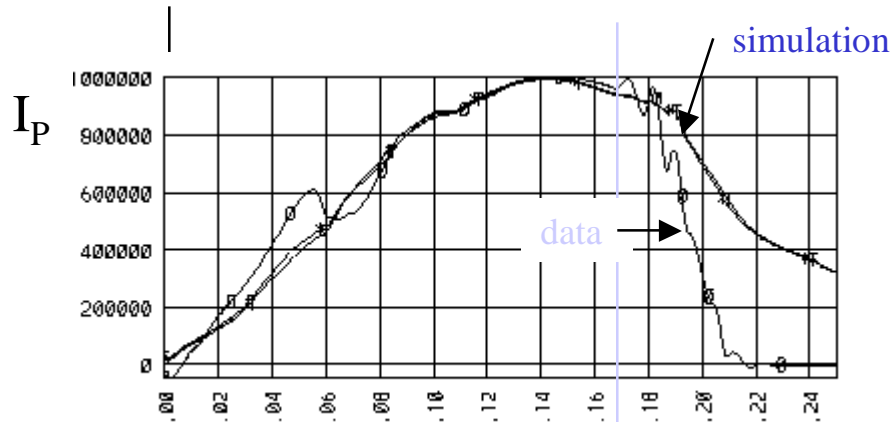


1MA ohmic shot 101522-- TSC results using measured coil currents vs data

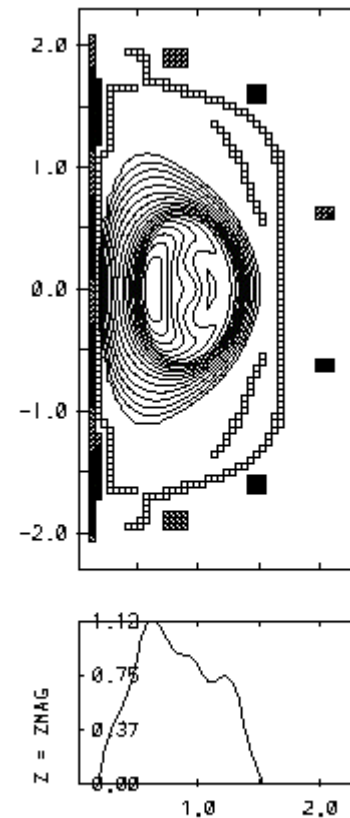
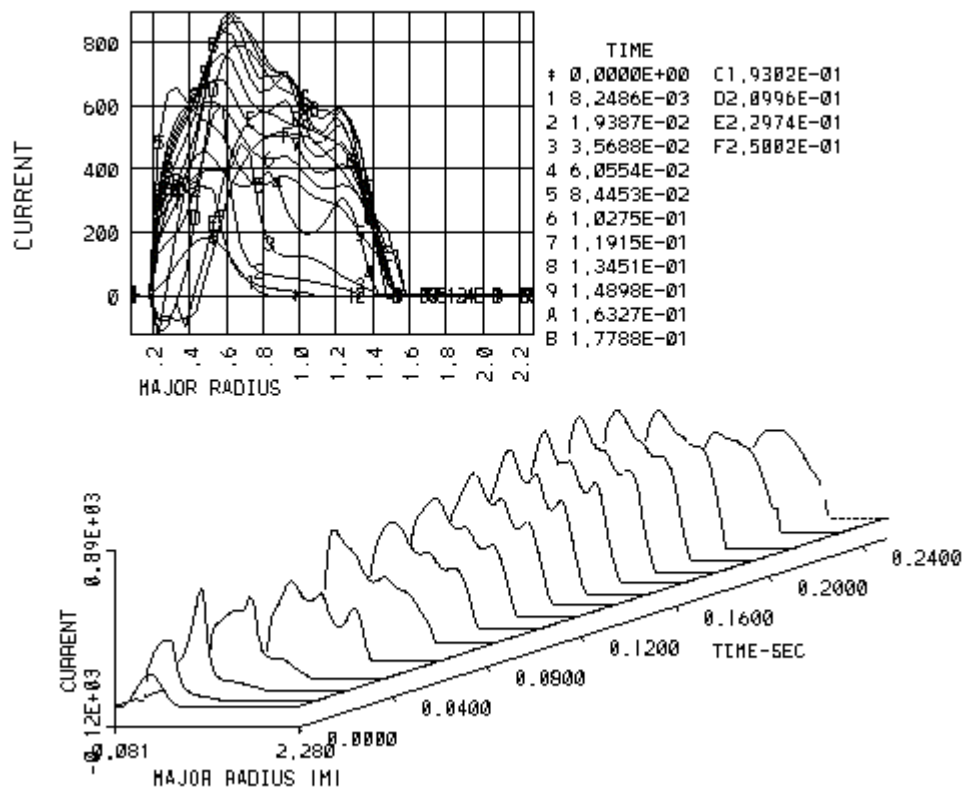


Comparison of flux loop values:
 green...TSC simulation
 red....Experiment

1MA ohmic shot 101522



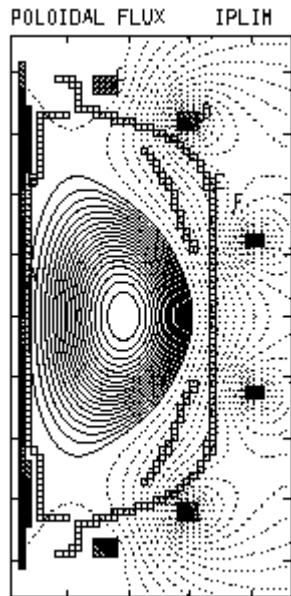
1 MA ohmic shot 101522



Note: plasma current not equilibrated

TIME = 177.88 MS CYCL

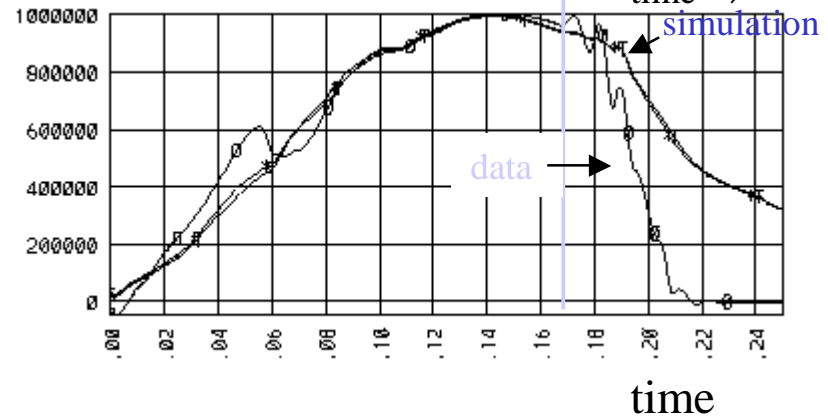
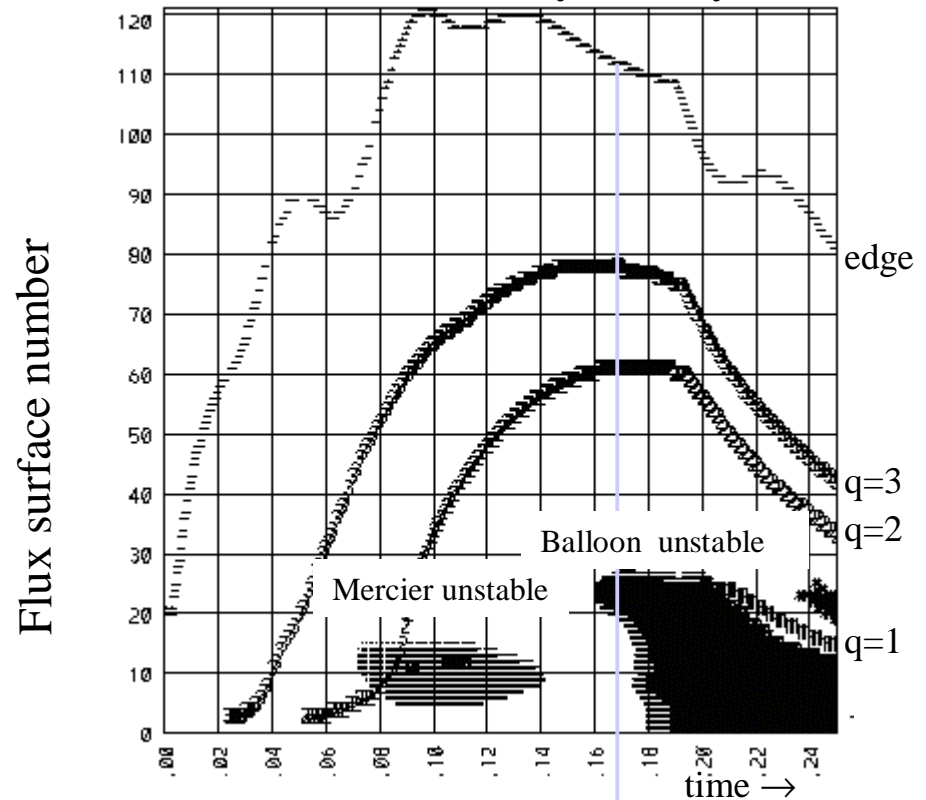
1 MA ohmic shot 101522



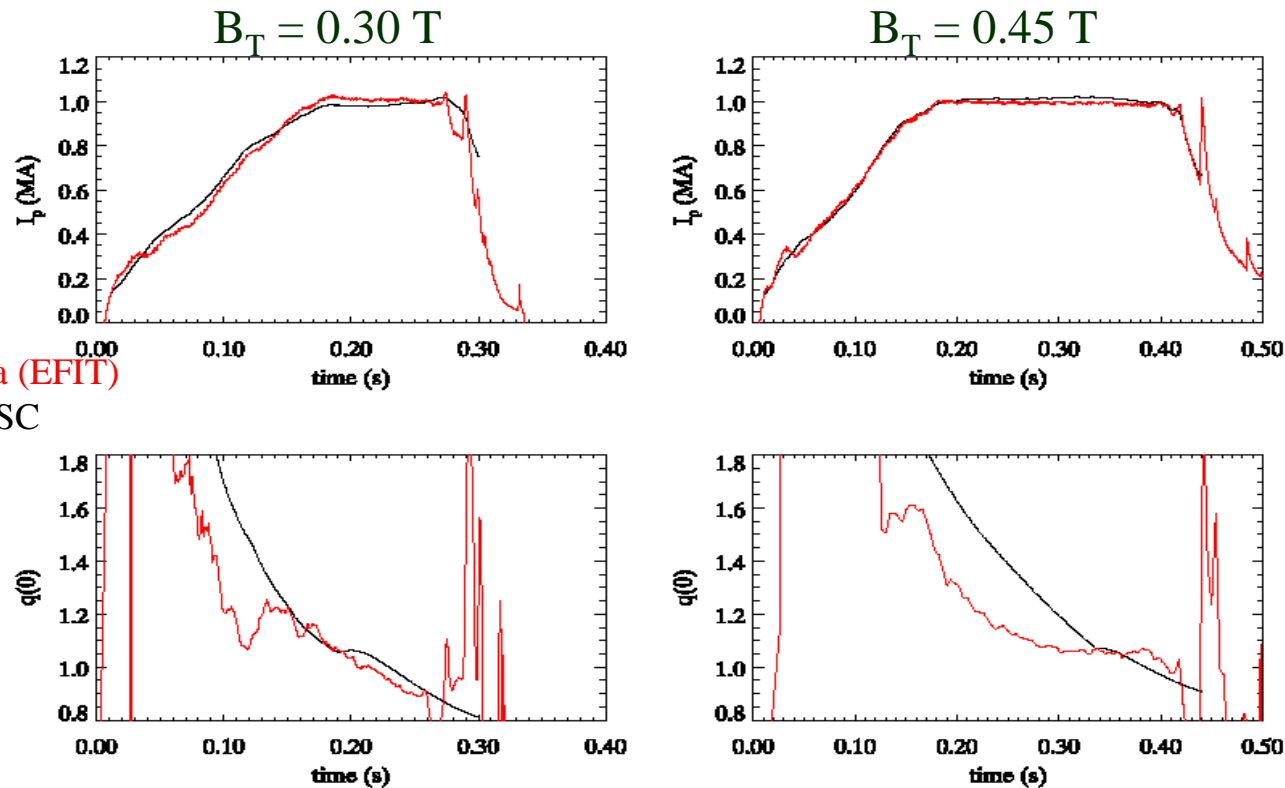
I_p

Large region of Balloon instability develops about time discharge “terminates”

Balloon Stability History



TSC was used recently to model the NSTX current evolution for a Toroidal Field scan series with 1.5MW beams. (Gates, et al)

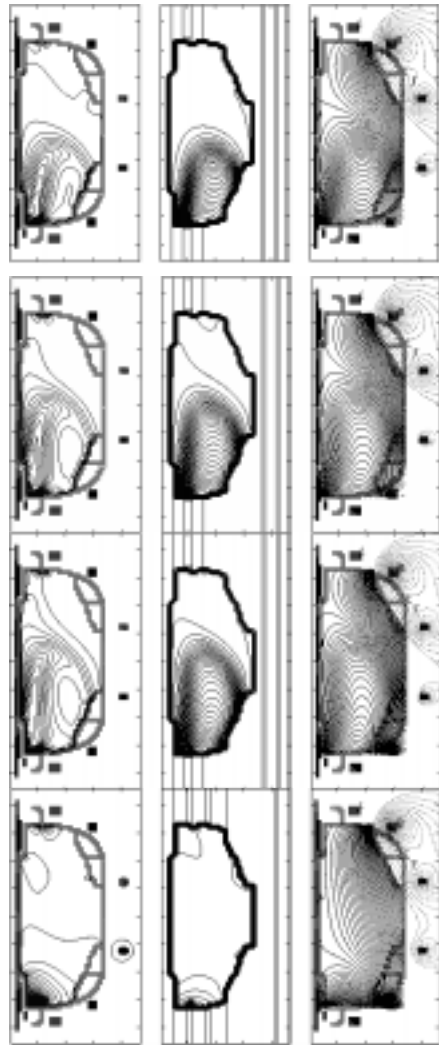


Red = data (EFIT)

Black = TSC

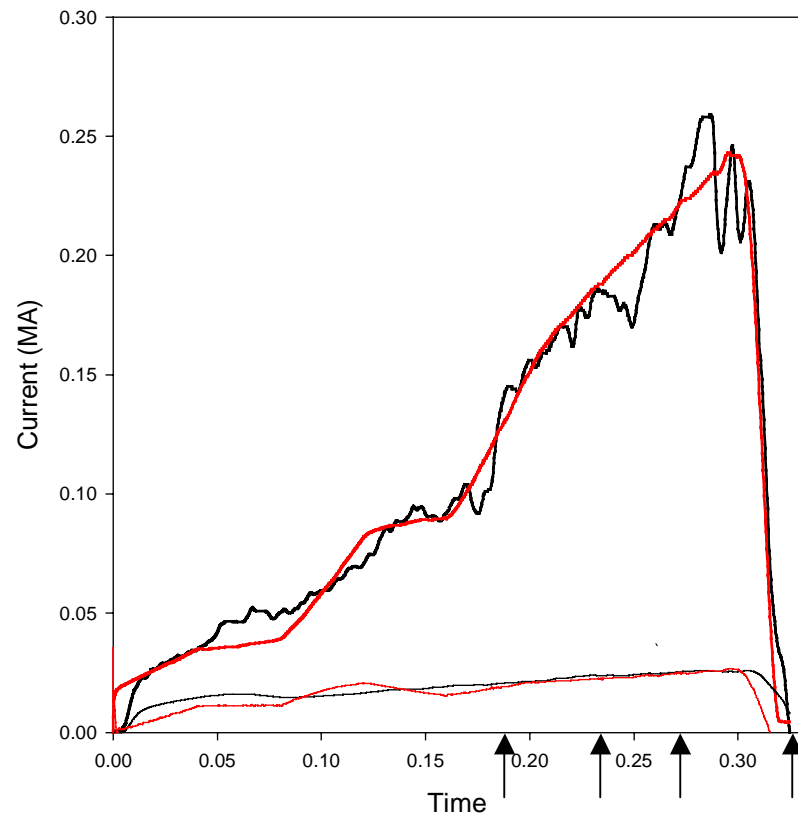
- TSC could reproduce the plasma current evolution using only the experimental values of the PF current trajectories. Everything else is predictive
- Supported the correlation between the $q=1$ surface and termination of the current

Toroidal current poloidal current poloidal flux

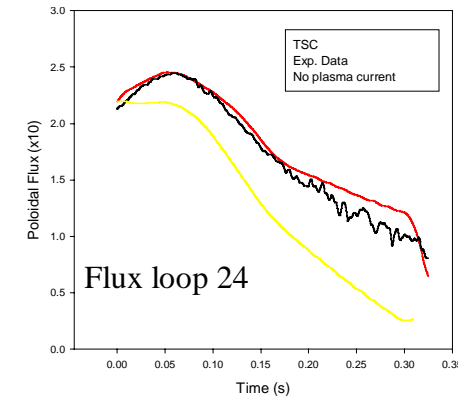
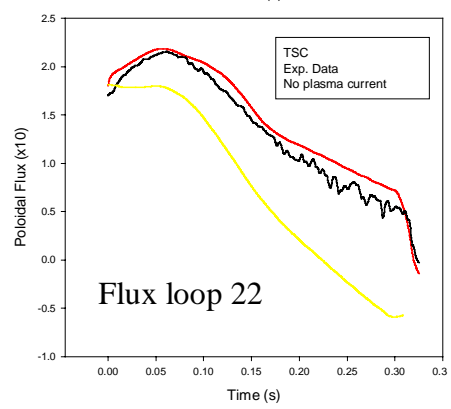
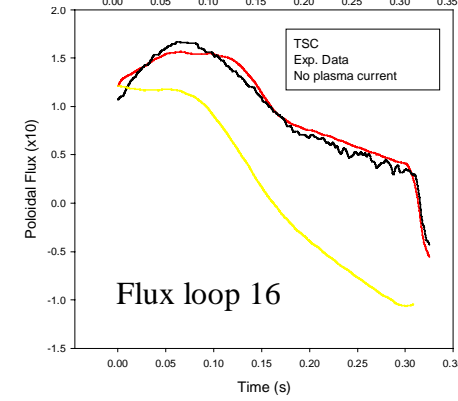
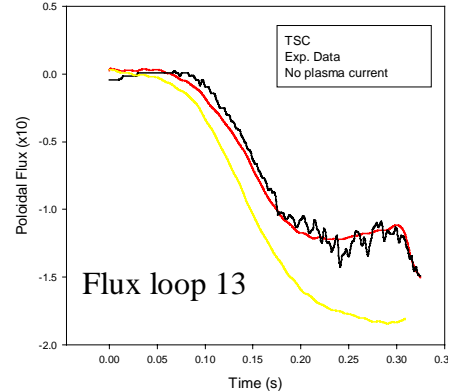
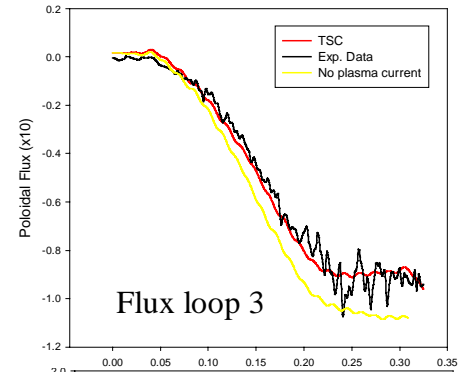
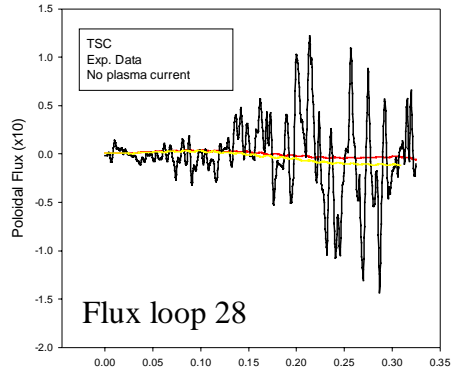
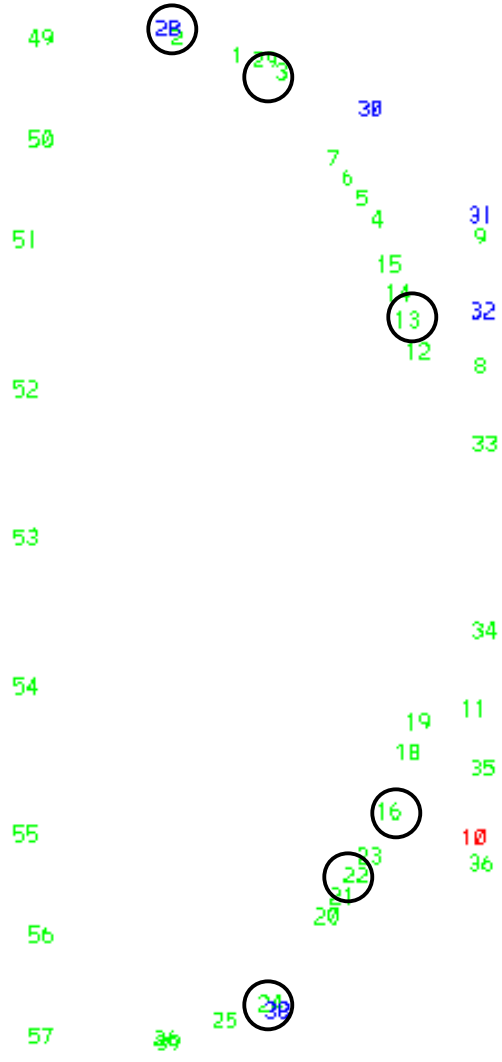


NSTX CHI shot 105513

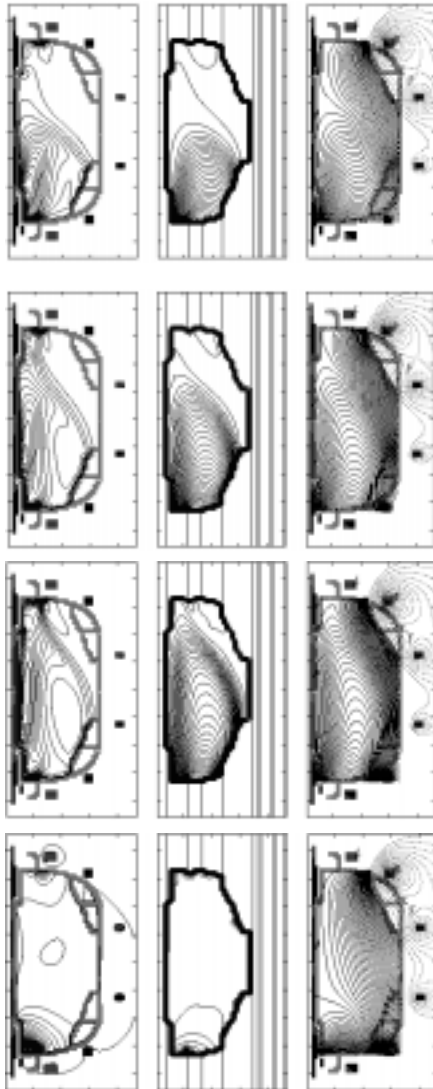
CHI shot 105513 --- currents



105513

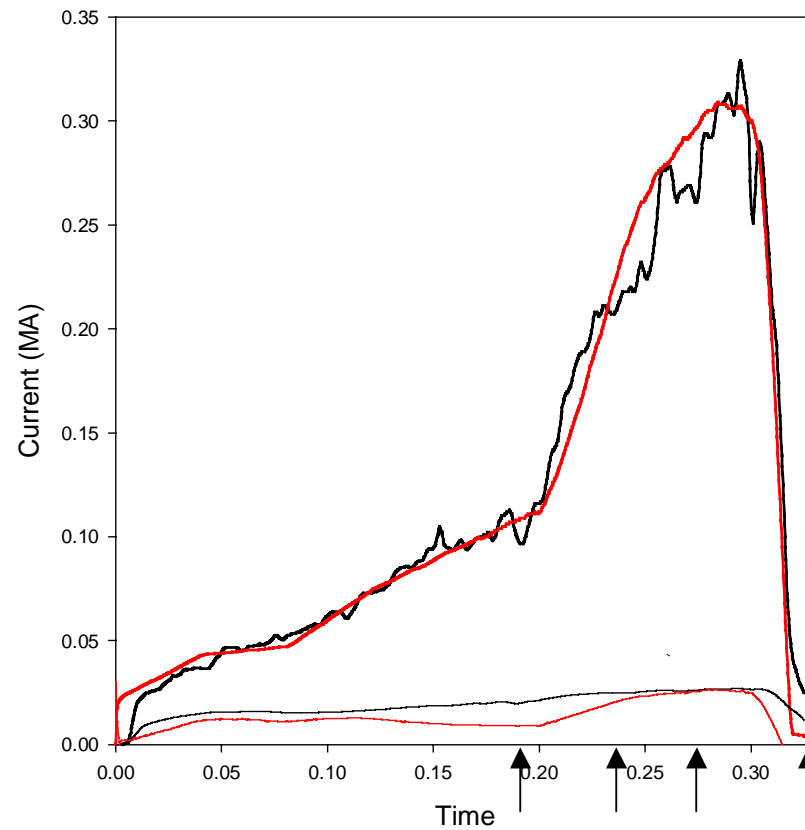


Toroidal current poloidal current poloidal flux

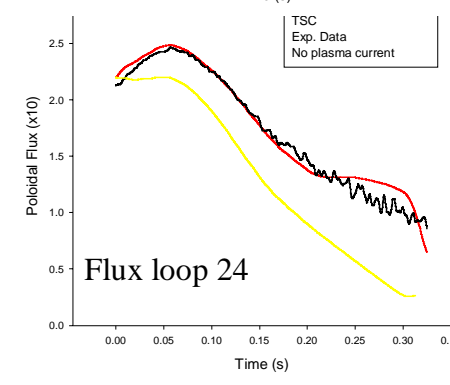
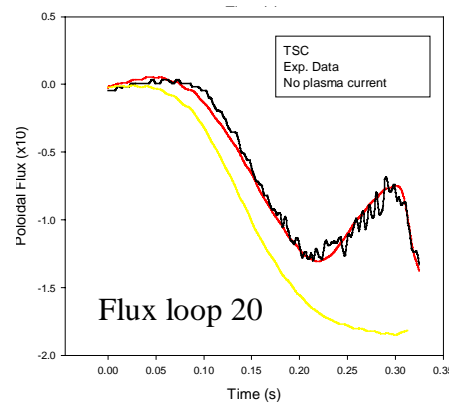
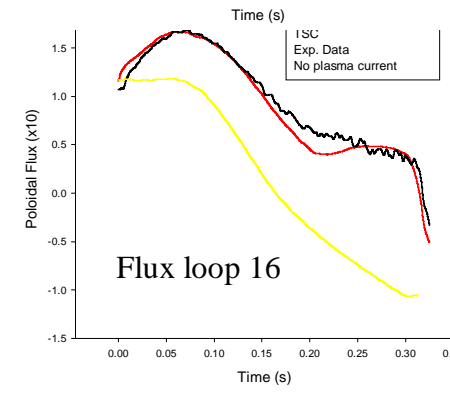
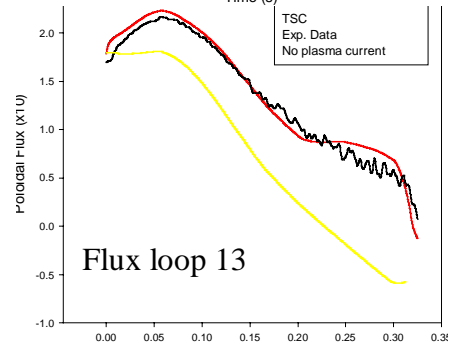
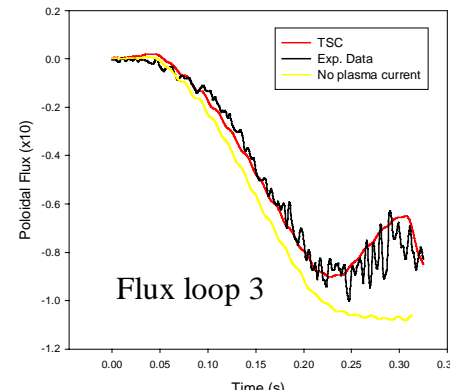
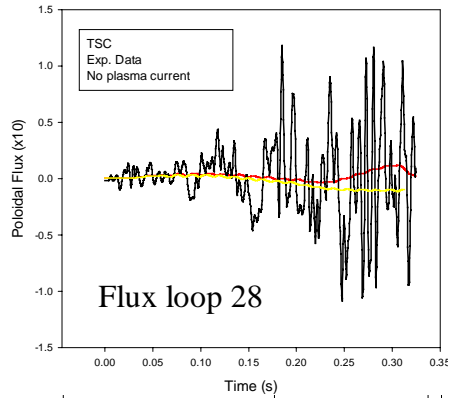
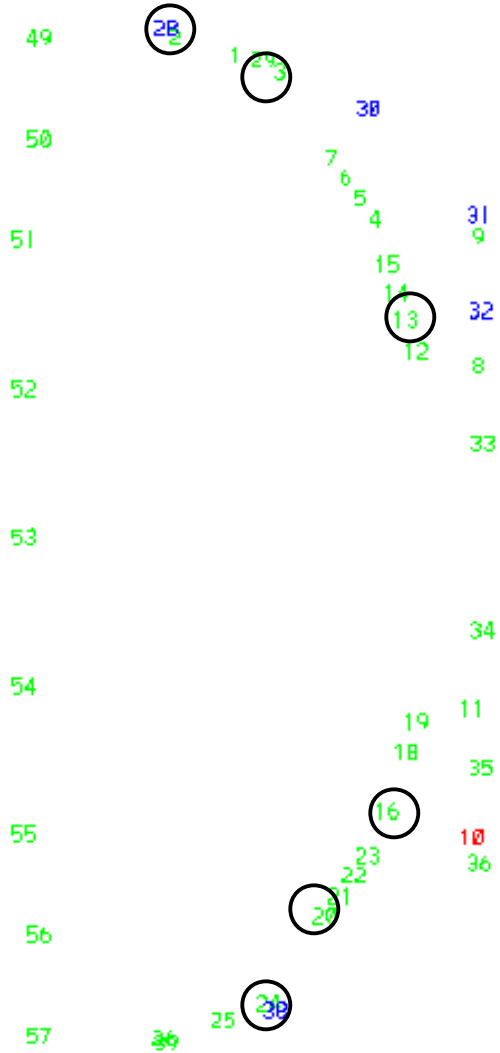


NSTX CHI shot 105514

CHI shot 105514 --- currents



105514



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