

# Analysis of Fission Neutron Spectra of $^{235}\text{U}$ and $^{239}\text{Pu}$

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# Introduction

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- Nuclear data for nuclear applications
- Los Alamos (Madland-Nix) model
- Sparse data of  $^{235}\text{U}$  and  $^{239}\text{Pu}$  fission neutron spectra
- Comprehensive measurements using white neutrons at WNR/LANSCE

# Experiments

- FIGARO
- CEA fission chamber
  - $^{235}\text{U}$  100 mg
  - $^{239}\text{Pu}$  90 mg
- Methods
  - Double time-of-flight technique
  - Pulse-shape discrimination
- Fake-background measurement
- Calibration



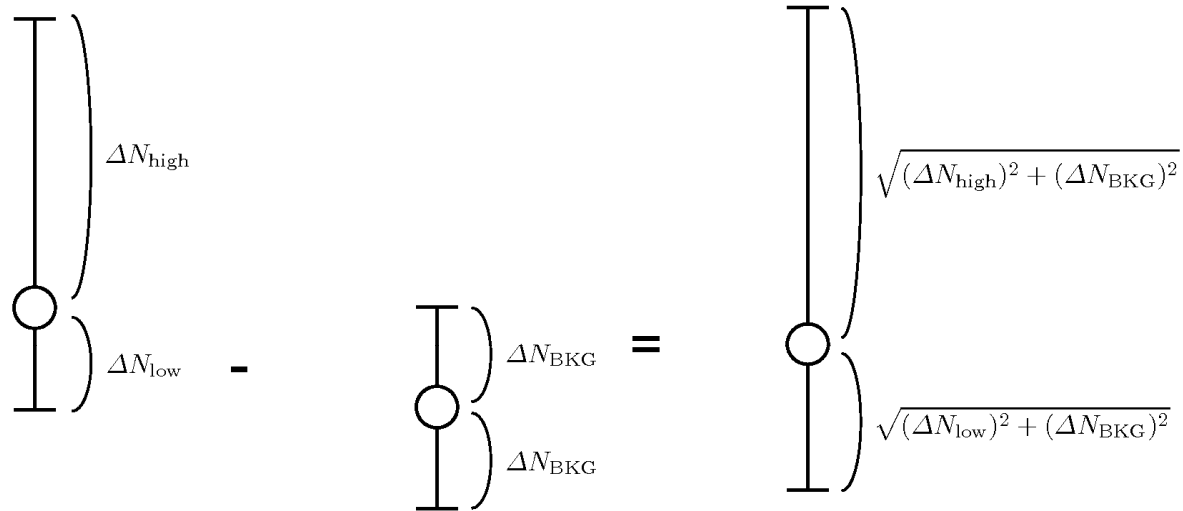
# Analysis

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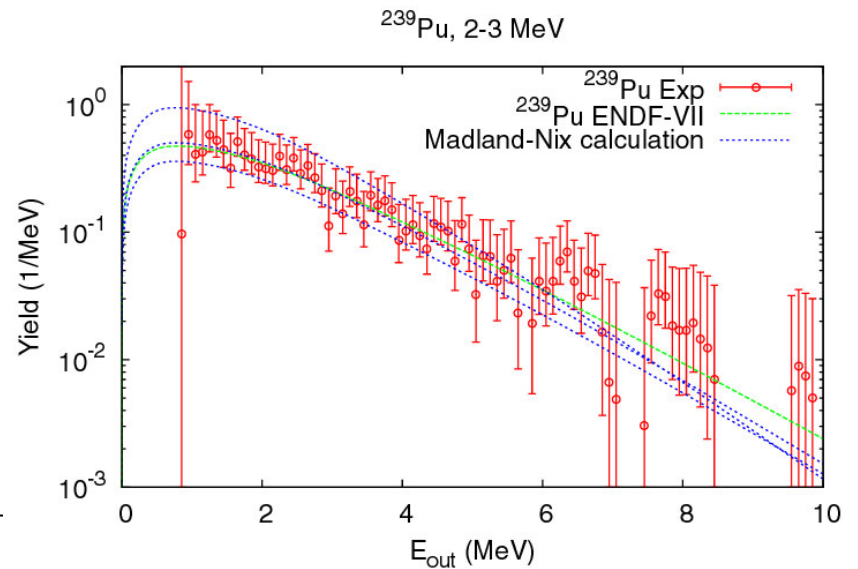
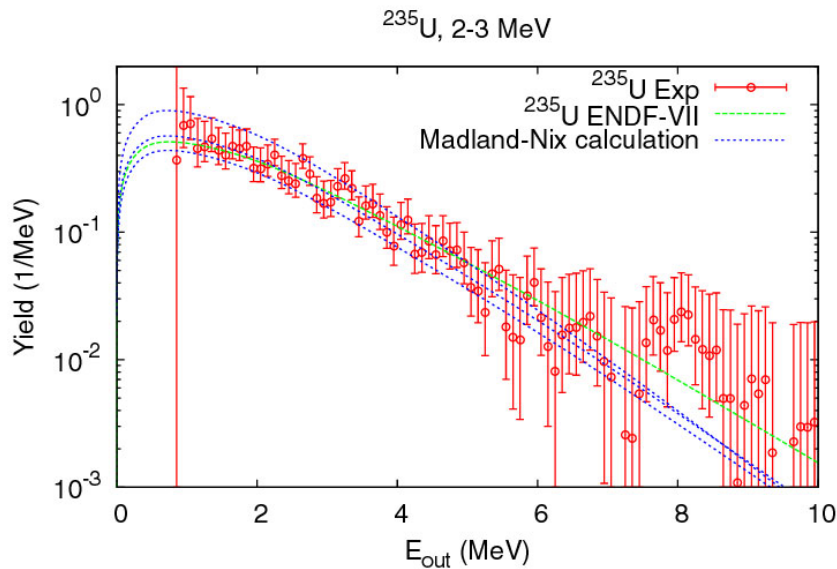
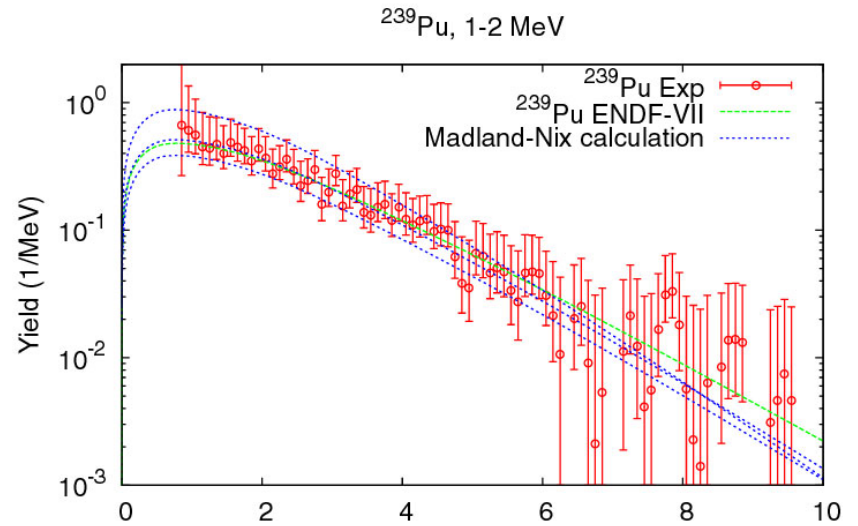
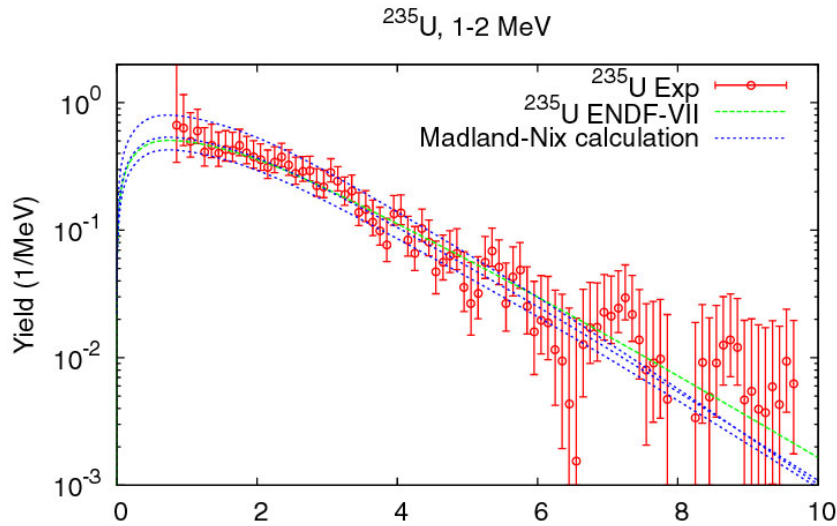
- Madland-Nix model calculation
  - Total kinetic energy (tke) parameter adjustment
- Average energy
  - Experimental results
  - ENDF-VII evaluated data
  - Madland-Nix model calculations

# Errors

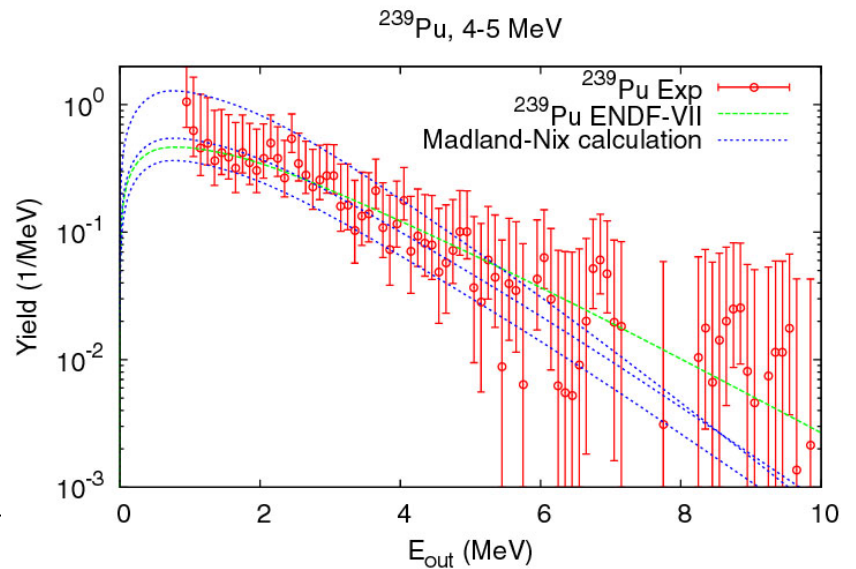
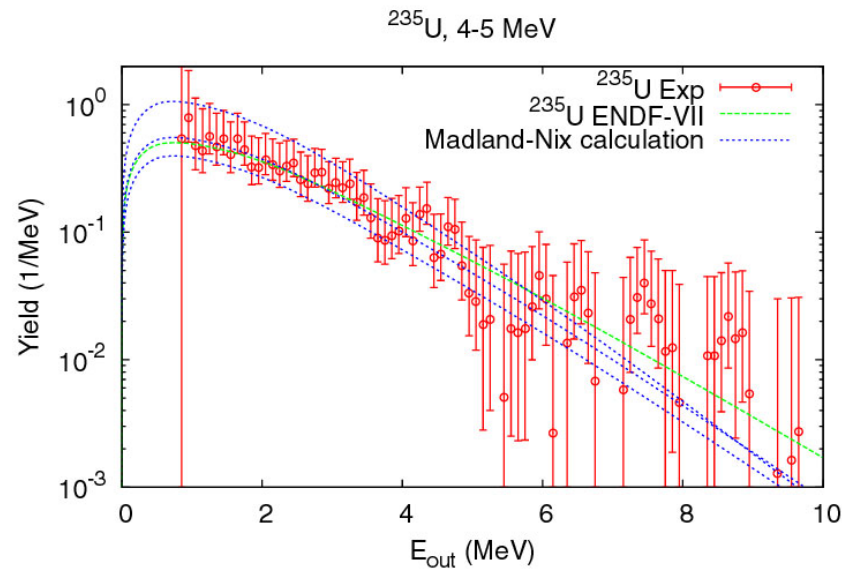
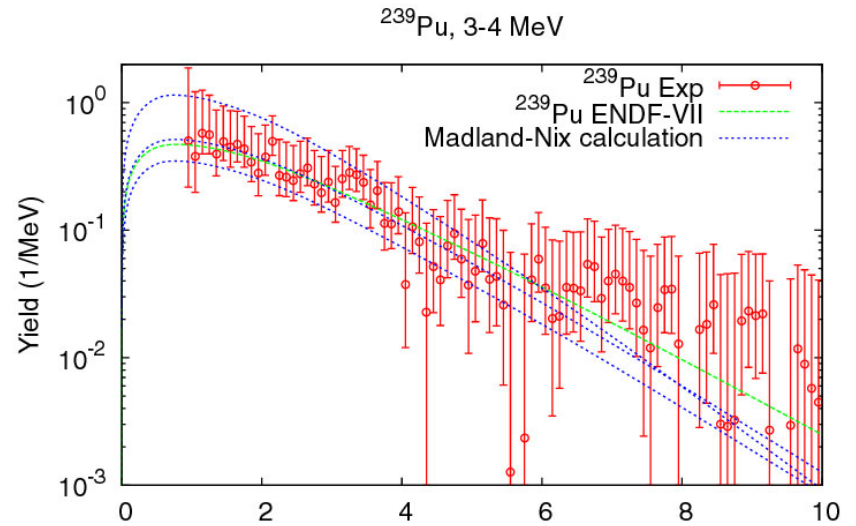
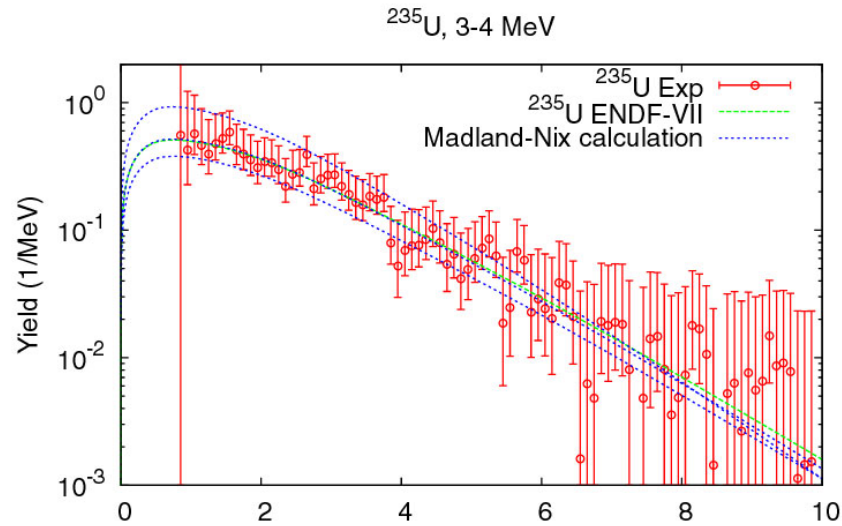
- Poisson statistic for fore-ground
  - Poor statistic
- Gaussian statistic for fake-background
- Assume the error-propagation



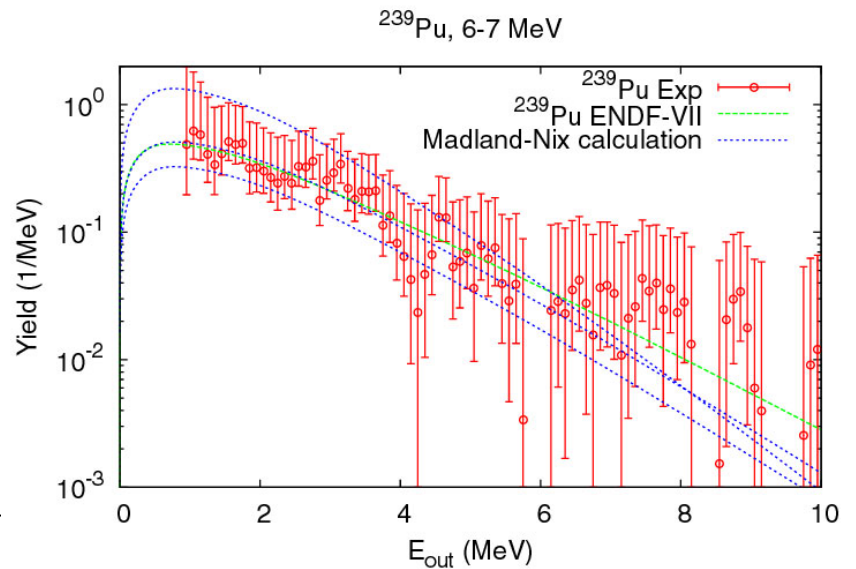
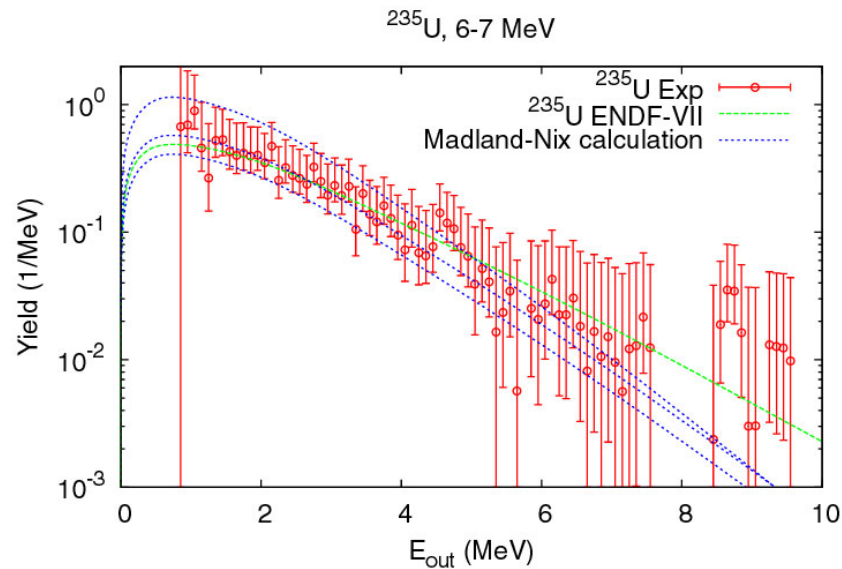
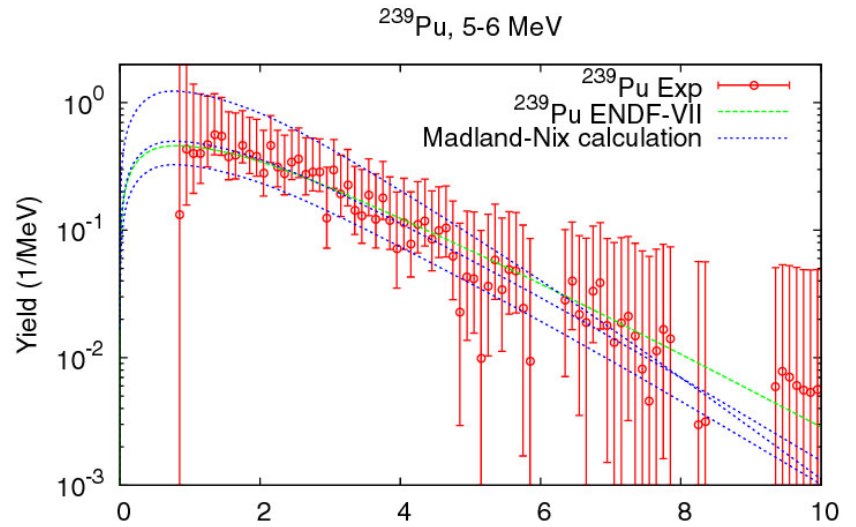
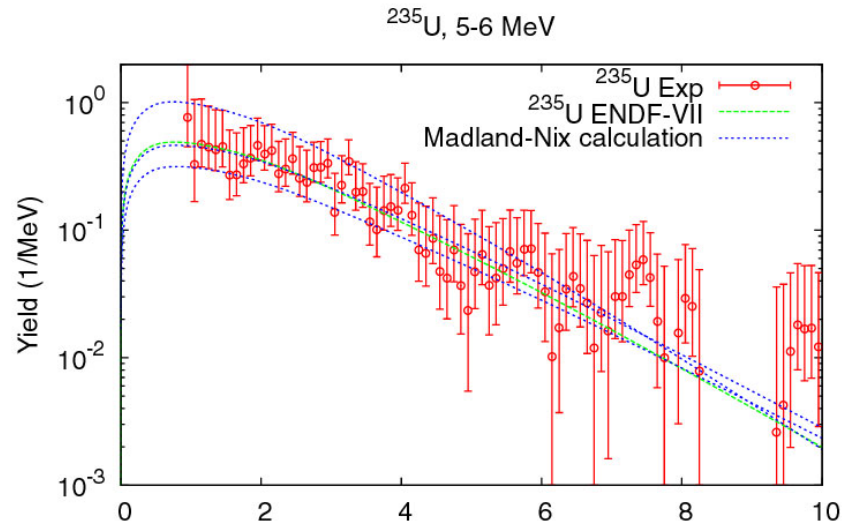
# Results, Fission neutron spectra, 1-2 & 2-3 MeV



# Results, Fission neutron spectra, 3-4 & 4-5 MeV

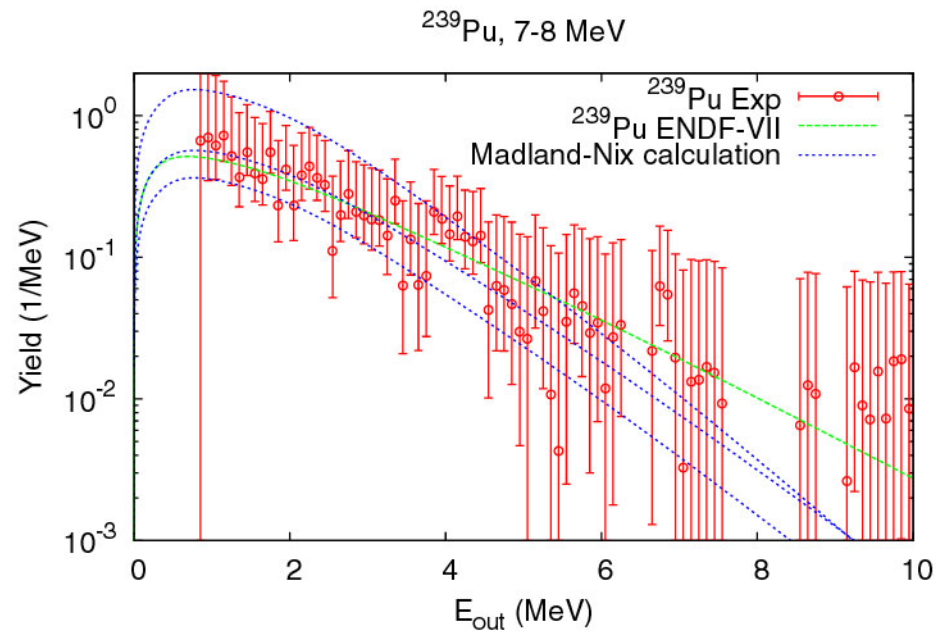
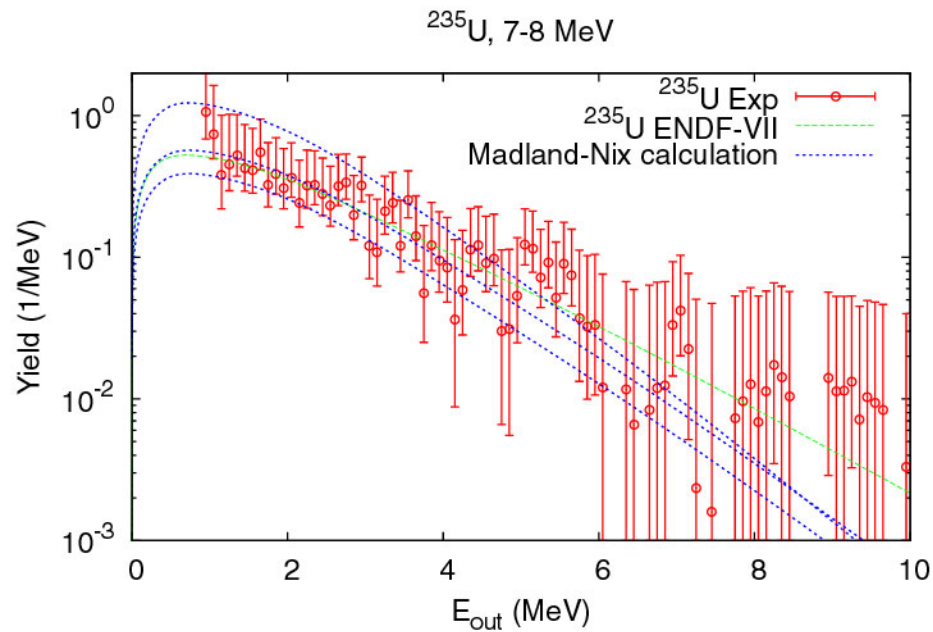


# Results, Fission neutron spectra, 5-6 & 6-7 MeV

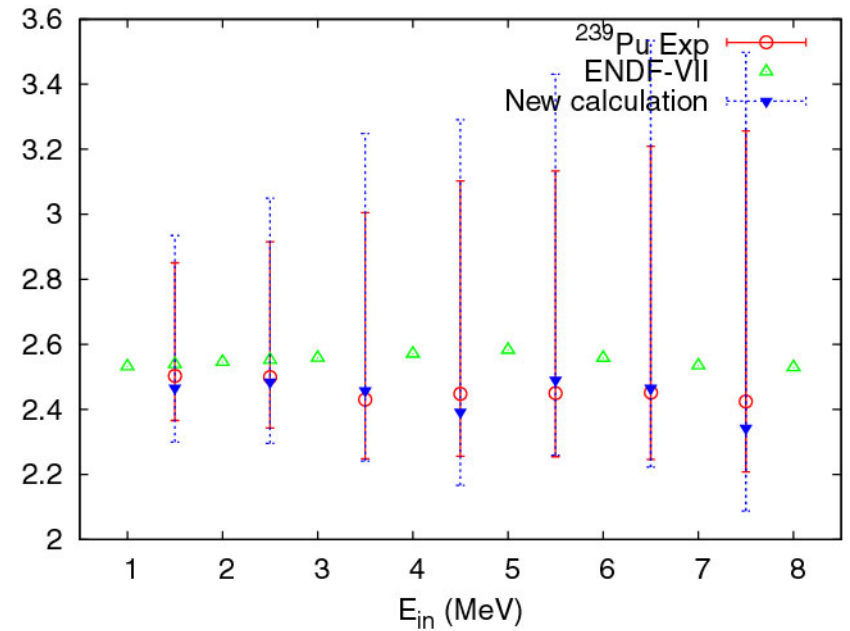
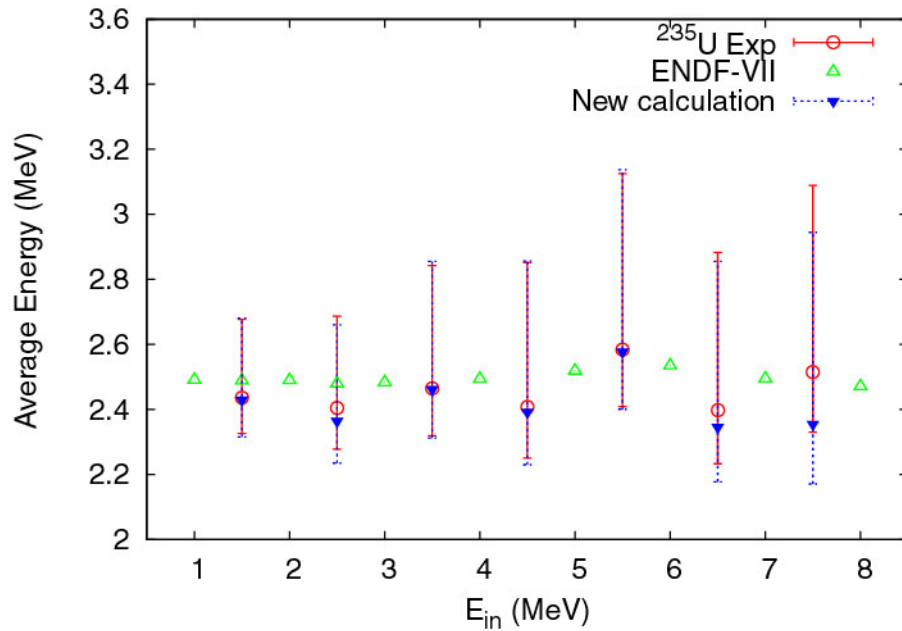




# Results, Fission neutron spectra, 7-8 MeV



# Results, Average energy



# Conclusion

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- Measured the fission neutron spectra of  $^{235}\text{U}$  and  $^{239}\text{Pu}$  for 1-2, 2-3, ..., 7-8 MeV inc
- Calculate average energies of  $^{235}\text{U}$  and  $^{239}\text{Pu}$  for the energy range 1-8 MeV inc
  - Experimental results
  - ENDF-VII evaluated data
  - Madland-Nix model calculations