

1200 New Jersev Ave.. COMPETENT AUTHORITY CERTIFICATION Washington, D.C. 20590 FOR A TYPE B(U)

East Building, PHH-23

Pipeline and Hazardous Materials

RADIOACTIVE MATERIALS PACKAGE DESIGN Safety Administration CERTIFICATE USA/6788/B(U) -96, REVISION 7

REVALIDATION OF UNITED KINGDOM COMPETENT AUTHORITY CERTIFICATE GB/2799E/B(U)-96

This certifies that the radioactive material package design described is hereby approved for use within the United States for import and export shipments only. Shipments must be made in accordance with the applicable regulations of the International Atomic Energy Agency¹ and the United States of America².

- Package Identification 2799E Safkeg. 1.
- Package Description and Authorized Radioactive Contents as described in United Kingdom Certificate of Competent Authority GB/2799E/B(U)-96, Issue 2 (attached). Contents are restricted to nonfissile or fissile excepted material.
- 3. General Conditions
 - a. Each user of this certificate must have in his possession a copy of this certificate and all documents necessary to properly prepare the package for transportation. The user shall prepare the package for shipment in accordance with the documentation and applicable regulations.
 - b. Each user of this certificate, other than the original petitioner, shall register his identity in writing to the Office of Hazardous Materials Technology, (PHH-23), Pipeline and Hazardous Materials Safety Administration, U.S. Department of Transportation, Washington D.C. 20590-0001.
 - c. This certificate does not relieve any consignor or carrier from compliance with any requirement of the Government of any country through or into which the package is to be transported.
 - d. This certificate provides no relief from the limitations for transportation of plutonium by air in the United States as cited in the regulations of the U.S. Nuclear Regulatory Commission 10 CFR 71.88
 - e. Records of Quality Assurance activities required by Paragraph 310 of the IAEA regulations shall be maintained and made available to authorized officials for at least three years after the last shipment authorized by this certificate. Consignors exporting shipments from the United States shall satisfy the requirements of Subpart H of 10 CFR 71.

^{1 &}quot;Regulations for the Safe Transport of Radioactive Materials, 1996 Edition (Revised), No. TS-R-1 (ST-1, Revised)", published by the International Atomic Energy Agency (IAEA), Vienna, Austria.

² Title 49, Code of Federal Regulations, Parts 100 - 199, United States of America.

CERTIFICATE USA/6788/B(U)-96, REVISION 7

4. Special Conditions -

- a. The Model No. 2799E containment boundary, which is provided by the Model No. 2812 containment can, must be tested to demonstrate that the leak rate does not exceed 1 x 10^{-8} Pa m^3 s^{-1} standardized leak rate (or 1 x 10^{-7} std cm³/s) at the following intervals: (1) within the 12-month period prior to shipment; and (2) upon replacement of the seals.
- b. Prior to each shipment, the seals of the Model No. 2812 containment can must be demonstrated to prevent leakage in excess of 5 x 10^{-5} Pa m³ s⁻¹ standardized leak rate (or 5 x 10^{-4} ref cm³/sec).
- c. This package is authorized for transport by air. For import to, export from, domestic shipments within the United States, and transport by aircraft through the air space of the United States from a foreign nation to a foreign nation, the maximum quantity of material transported by air may not exceed an A_2 quantity of plutonium in any isotope or form, as defined in 49 CFR 173.435.
- Marking and Labeling The package shall bear the marking USA/6788/B(U)-96 in addition to other required markings and labeling.
- Expiration Date This certificate expires on June 30, 2010.

This certificate is issued in accordance with paragraph 808 of the IAEA Regulations and Section 173.473 of Title 49 of the Code of Federal Regulations, in response to the March 12, 2008 petition by Croft, Inc., Los Alamos, NM, and in consideration of other information on file in this Office.

Certified by:

MAY 27 2008

(DATE)

obert A. Richard

Deputy Associate Administrator for Hazardous Materials Safety

Revision 7 - issued to endorse, for non-fissile and fissile excepted contents, United Kingdom Certificate of Approval No. GB/2799E/B(U)-96, Issue 2 and to restrict air transport.



Issue

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Page 1 of 7 pages

Certificate of Approval of Package Design for the Carriage of Radioactive Materials

THIS IS TO CERTIFY that the Secretary of State for Transport being, for the purposes of the Regulations of the International Atomic Energy Agency, the Competent Authority of Great Britain in respect of inland surface transport and of the United Kingdom of Great Britain and Northern Ireland in respect of sea and air transport and the Department of the Environment for Northern Ireland being the Competent Authority of Northern Ireland in respect of inland surface transport, have approved the Package design as specified in section 1 of this certificate, as applied for by Croft Associates Ltd (see section 6)

as Type B(U)

by All modes

Packaging identification: Safkeg

Packages manufactured to this design meet the requirements of the regulations and codes on page 2, relevant to the mode of transport, subject to the following general condition and to the conditions in the succeeding pages of this certificate.

In the event of any alteration in the composition of the package, the package design, the quality assurance programme(s) associated with the package or in any of the facts stated in the application for approval, this certificate will cease to have effect unless the Competent Authority is notified of the alteration and the Competent Authority confirms the certificate notwithstanding the alteration.

Expiry Date: This certificate is valid until the end of June 2010 (see section 6)

COMPETENT AUTHORITY IDENTIFICATION MARK:

Type B(U) GB/2799E/B(U)-96

UNITED KINGDOM COMPETENT AUTHORITY FOR THE TRANSPORT - 5 MAR 2008

OF RADIOACTIVE MATERIALS

Transport Radiological Adviser
Department for Transport
Great Minster House
76 Marsham Street
London SW1P 4DR

On behalf of the Secretary of State for Transport, and the Department of the Environment for Northern Ireland

This certificate does not relieve the consignor from compliance with any requirement of the government of any country through or into which the package will be transported.

Issue

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REGULATIONS AND CODES OF PRACTICE GOVERNING THE TRANSPORT OF RADIOACTIVE MATERIALS

INTERNATIONAL

International Atomic Energy Agency (IAEA)

TS-R-1 Regulations for the Safe Transport of Radioactive Materials 2005 Edition.

International Maritime Organisation (IMO)

International Maritime Dangerous Goods (IMDG) Code Amendment 33-06.

International Civil Aviation Organisation (ICAO)

Technical Instructions for the Safe Transport of Dangerous Goods by Air 2007-2008 Edition.

United Nations Economic Commission for Europe (UNECE)

European Agreement concerning the International Carriage of Dangerous Goods by Road (ADR) 2007 Edition.

Intergovernmental Organisation for International Carriage by Rail (OTIF)

Convention concerning International Carriage by Rail (COTIF) Appendix B. Uniform Rules concerning the Contract for International Carriage of Goods by Rail (CIM) Annex 1 Regulations concerning the International Carriage of Dangerous Goods by Rail (RID) 2007 Edition.

UNITED KINGDOM

ROAD

GREAT BRITAIN ONLY.

The Carriage of Dangerous Goods and Use of Transportable Pressure Equipment Regulations 2007, SI 2007 No 1573.

NORTHERN IRELAND ONLY.

The Radioactive Substances (Carriage by Road) Regulations (Northern Ireland) 1983, SR 1983 No 344. The Radioactive Substances (Carriage by Road) (Amendment) Regulations (Northern Ireland) 1986, SR 1986 No 61.

RAIL

GREAT BRITAIN ONLY.

The Carriage of Dangerous Goods and Use of Transportable Pressure Equipment Regulations 2007, SI 2007 No 1573

SEA

British registered ships. All other ships whilst in United Kingdom territorial waters. The Merchant Shipping (Dangerous Goods and Marine Pollutants) Regulations 1997, Si 1997 No 2367; Merchant Shipping Notice No MSN 1806 M, "The Carriage of Dangerous Goods and Marine Pollutants in Packaged Form - Amendment 33-06 to the International Maritime Dangerous Goods (IMDG) Code".

AIR

The Air Navigation Order 2005, SI 2005 No 1970. The Air Navigation (Dangerous Goods) Regulations 2002, SI 2002 No 2786. The Air Navigation (Dangerous Goods) (Amendment) Regulations 2007, SI 2007 No 28.

Issue 2

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1. PACKAGE DESIGN SPECIFICATION

The Package Design Specification shall be in accordance with Croft Associates Ltd Safety Report reference DSR 2799E-96 Issue D dated 4 March 2008 and modifications to the package design approved by the authority named on page 1 of this certificate under the established modifications procedure.

1.1 Specification of Design

Design No.	Title (number of components)	Drawing / Drawing List	Issue	
2799	Outer / Keg (one)	DI 40 5407		
2812	Inner / Containment can (one)	DL-1C-5407	С	

1.2 Authorised Contents

- a) Generalised content type. Uranium, plutonium, americium, thorium, californium, neptunium, strontium and curium, compounds and/or mixtures or Pu/Be neutron sources. Fissile material is limited to fissile exempt quantities. Non-fissile alpha, beta, gamma emitting isotopes and small neutron emitting sources. As specified in PCS 008 Issue S.
- b) The maximum mass of radioactive contents is 10,000g for U-238, Np-237, Sr-90 and Cm-244 but is less for other nuclides see PCS 008 issue S.
- c) The physical form of the nuclides shall be solids as powder, dried cake, sintered pellets or metal pieces.
- d) The chemical form of the nuclides shall be either elemental or compound form.
- e) The package may be transported empty of contents.
- f) The total rate of heat generation shall not exceed 10W
- g) The total activity of the content shall not exceed 55.2TBq for Sr-90 but is less for other nuclides see PCS 008 issue S.
- h) The isotopes shall be contained in primary containers
- i) The powders carried shall have been produced by normal manufacturing techniques and not have been further processed to specifically produce fine powders.

1.3 Fissile Material Restrictions

a) The contents of the package and/or consignment meet the requirements of paragraph 672 of IAEA TS-R-1 (2005 Edition).

1.4 Package Dimensions and Weights

a) Nominal Dimensions: 425mm diameter x 540mm long (see section 5 for package illustration)

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b) Maximum authorised gross weight: 67.5kg

2. USE OF PACKAGE

2.1 Use of packaging

a) The packaging shall be used, handled and maintained in accordance with CPI 013 Issue N (Packing and Handling Instructions for Package Design No 2799E) and CSP 006 Issue H (Serviceability checks on Package Design No 2799E).

2.2 Supplementary Operational Controls

- a) The accessible surfaces of the package must not exceed 85°C. Suitable measures (such as the use of a light metal canopy to prevent access to the package surface) must be taken to ensure that no access can be gained to surfaces with a temperature in excess of 85°C.
- b) The package shall not be sheeted over or over-stowed by loose cargo

2.3 Actions prior to shipment

a) Administrative controls shall ensure that the contents are in accordance with section 1 of this certificate, and that the consignor and consignee hold a copy of the instructions on the use of the packaging.

2.4 Emergency Arrangements

- a) Before shipment takes place, the consignor shall have drawn up suitable emergency plans, copies of which shall be supplied to the UK Competent Authority on demand.
- b) Within the UK the RADSAFE scheme will be used.
- c) If the consignor's own, or other approved emergency plans cannot be initiated, for any reason, then the police shall be informed immediately and requested to call the local NAIR (National Arrangements for Incidents involving Radioactivity) establishment.

2.5 Ambient temperature range for package design

a) -40°C to +38°C

3. QUALITY ASSURANCE

- 3.1 Quality assurance programmes applicable to this design are:
 - a) Croft Associates Ltd Quality Management System.
 - b) any other quality assurance programmes associated with the design, manufacture, testing, documentation, use, maintenance and inspection, and for transport and in-

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transit storage operations, which must also comply with national or international standards for quality assurance which are acceptable to the authority named on page 1 of this certificate.

- 3.2 No alterations shall be made to the quality assurance programmes associated with this design and approved by the authority named on page 1 of this certificate unless that alteration has the prior approval of said authority, or it falls within the agreed change control procedures of that programme.
- 3.3 No quality assurance programme shall be used at any stage of the design, manufacture, testing, documentation, use, maintenance and inspection, and for transport and in-transit storage operations, unless said programme forms part of or is the quality assurance programme approved by the authority named on page 1 of this approval certificate.

4. ADMINISTRATIVE INFORMATION

- 4.1 Other related certificates (alternative radioactive contents)
 - a) This certificate forms the base approval of this design. Other related UK certificates using the 2799 outer are shown below: -

Certificate Reference & Issue	Certificate	Expiry Date
	Type	
None currently valid		

The list in 4.1(a) was complete at the time of compilation of this design approval certificate. Other related certificates may exist.

4.2 Additional Technical Data / Information

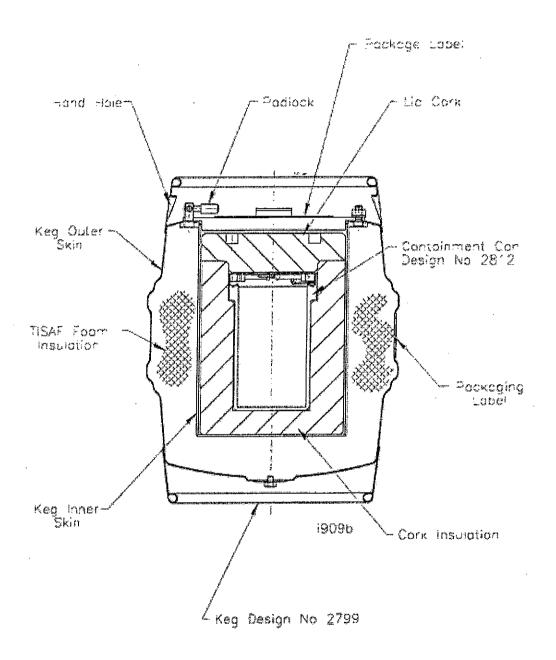
a) At the time of compilation of this design approval certificate, The Ionising Radiations Regulations 1999, SI 1999 No 3232 and Approved Code of Practice apply, with regard to radiation protection, to all modes of transport and The Dangerous Substances in Harbour Areas Regulations 1987, SI 1987 No 37, apply in UK Ports

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5. PACKAGE ILLUSTRATION



Issue

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6. CERTIFICATE STATUS

Design Approval issued to:-Croft Associates Ltd

Croft Associates Ltd F4 Culham Science Centre Abingdon Oxon OX14 3DB United Kingdom

Issue No.	Date of Issue	Date of Expiry	Reason for Revision
GB/2779E/B(U) –96 Issue 2	See pg 1 of this certificate	End of June 2010	Renewal with minor changes
GB/2779E/B(U)96 Issue 1	24 June 2004	End of June 2007	Design approval issued under new regulations; Fissile contents removed; and minor amendments/clarification.
GB/2799E/B(U)F-85 Issue 4	18 June 2001	31 March 2004	Renewal.



Package Contents Specification for

Package Design No 2799E

	Package Contents	Number	PCS 008				
Title	Specification for Package	Issue	S				
	Design No 2799E	File Reference	PCS 008-R-96-Package Contents Specification 2799E				
Compiled	S H Marshall	Checked	A L Ferguson				
Approved	R P Hows	Date	05 Dec 07				

1 Introduction

The 2799E package design is one of the Croft SAFKEG series of packages that all have an outer container that uses a stainless steel keg (similar to those used in beer kegs) which is provided with a stainless steel liner, the interspace being filled with TISAF (Thermal Insulating and Shock Absorbing Foam – a phenolic resin blown foam). The outer keg carries a single resealable, leaktestable stainless steel containment vessel within cork packing.

The 2799E package is a general purpose container for the shipment of non-fissile and fissile excepted material in solid form. The packaging consists of a Keg Assembly Design No 2799, carrying a single Containment Vessel (CV) Assembly Design No 2812 within an insulating cork liner.

This report assesses the limiting criterion for the radionuclides to be transported as detailed in table 2, to ensure compliance with the IAEA TS-R-1 transport Regulations [Ref 1], and identifies the package limit for each nuclide.

2 Specification of Contents

2.1 General nature of content

The contents may be non fissile or fissile excepted alpha, beta or gamma emitting isotopes, uranium, plutonium, americium and thorium in compound form or mixtures. Or the contents may be neutron sources of Pu/Be, Am mixed with beryllium, boron, lithium or fluorine or Californium 252.

2.2 Radionuclides

See Table 2. Associated daughter products are also allowed, however, any daughter products that do not provide a significant contribution to the limits shown are not listed.

Maximum enrichment for U235 shall be 100% by weight.

2.3 Physical state

Solids as powder, dried cake, metal pieces or sintered pellets.

Powders shall have been produced by normal techniques and not have been further processed to produce specifically fine powder.

2.4 Chemical composition

Elemental or compound form.

2.5 Quantity

Maximum quantity of radioactive material to be carried is 10,000 g. The contents limit for the package (radioactive material + other materials + packing) is 11,500 g.

2.6 Activity Limit

2.6.1 Individual Radionuclide Limits

The package limits for individual nuclides for land, sea and air are given in Table 2 under the column headed "Package Limit".

2.6.2 Limits for Mixtures of Radionuclides

Where more than one nuclide is present, the contents are limited as below:

- Mixtures of the nuclides given in Table 1, which are individually limited by the Package Limit, are limited such that the sum of the proportionate amounts of each nuclide with respect to the quantity shown does not exceed unity.
- Content activity to be limited so that the total heat emission for a single package is less than 10 W
- Neutron emitters such as Cf 252 must not be carried with any fissile material
- Trace quantities of other radionuclides (other than those listed in Table 1) may be carried providing that their individual activities do not exceed 1% of the total activity of the contents.

2.7 Fissile Material Consignment limit [672]

For fissile material a mass limit per consignment will be applied such that:

$$\frac{\text{mass of uranium-235 (g)}}{X} + \frac{\text{mass of other fissile material (g)}}{Y} < 1$$

where X and Y are the mass limits defined below in table 1.

Fissile material	Fissile material mass (g) mixed with substances having an average hydrogen density less than or equal to water	Fissile material mass (g) mixed with substances having an average hydrogen density greater than water
Uranium 235	400	290
(X)		
Other fissile	250	180
material (Y)		

Table 1 – Consignment Mass Limits for Exceptions from the Requirements for Packages Containing Fissile Material (Ref 1)

However these limits may only be applied when:

- i. Each individual package contains not more than 15g of fissile material;
- ii. Neither beryllium nor deuterium shall be present in quantities exceeding 1% of the applicable consignment mass limits provided in Table 1.

2.8 Primary containers and packing

Primary containers may be polythene bottles and/or heat seal 'bags' which may then be placed in metal cans. The neutron sources of Am/(mixed with either Be, F, B or Li) and Cf 252 are packed in steel welded capsules composed of 2 or 3 welded envelopes.

If the solid radioactive material is (or may be) soluble in or leachable by water, a sachet of dry silica gel (not less than 10 g) must be packaged with the primary containers within the resealable stainless steel containment can (design No 2812).

3 Calculation of Allowable Content

3.1 Criticality limit

Contents are restricted to fissile excepted quantities.

3.2 Heat Limits

The heat limit has been set at 10W (1,690 MeV Ci). This heat load has been shown to produce acceptable temperatures in the packaging under the most restrictive assumptions required by the regulations.

3.3 Shielding Limit

The contents listed in Table 2 are primarily alpha or beta emitters for which the shielding provided by the stainless steel cans is sufficient to allow unlimited contents without external surface dose or TI (radiation level at 1 m from the external surface of the package) exceeding the regulatory limits.

The only nuclides which produce a significant gamma or neutron dose, either directly or from daughters, are 241Pu, 241Am, 243Am, 232U, Am/(mixed with either Be, F, B or Li), Cf and Pu/Be (sources). The quantities of these materials have been limited to ensure that the external radiation dose levels are within the regulatory limits.

When 243Am is carried a suitable lead pot must be used, see detailed instructions in CPI 013. The mass of contents and pot shall not exceed 11,500g as specified in 1.5 above.

3.4 Mass Limit

An arbitrary maximum mass for the package radioactive contents, and therefore any single nuclide, is set at 10,000 g. Lower mass limits for some nuclides are specified where these are more realistic.

241Pu is limited to 10% of the Pu limit as this is the upper limit for 241Pu in Pu (with a margin of safety).

232U is limited to 10-4 % of U limit as this is the upper limit for 232U in U (with a margin of safety).

3.5 Package Limit

The activity limits for individual nuclides for the different criteria of criticality, heat output, radiation shielding and mass, are as listed in Table 2. The maximum activity of each nuclide that may be carried in the package is chosen as the least of the separate limits listed in Table 2.

4 References

- 1. International Atomic Energy Agency, IAEA Safety Standards Series No TS-R-1, Regulations for the Safe Transport of Radioactive Material, 1996 Edition (As Amended 2003) and 2005 Edition
- 2. Croft Associates Ltd, CTR 83/3, Heat Generation by the decay of radioactive isotopes, Issue F.
- 3. Croft Associates Ltd, CS 2000/18, Package 2799E Dose Rates due to Neutron Emitters, Issue C
- 4. Croft Associates Ltd, CS 2000/19, Package 2799E Dose Rate and heat output from Americium-243, Issue A
- 5. Grove Engineering, USA, Microshield Version 6.02, Analysis of Shielding and exposure to gamma Radiation.

Fissile Nuclides			Specific			Heat	Limit	01 : 11:			Package limit			
FISSIIE NUCIIQES		TS-R-1	Activity	ctivity Criticality Limit		Heat Output	Activity for	y for Shielding	Mass limit				Air tra	ansport
Nuclide	Element	A ₂ value	[2]	[2]		[1]	10W	Limit			Land and S	ea transport	TS-R-1 Air limit (3000 /	
Nuclide	Element	(Bq)	(Bq/g)	(g)	(Bq)	(W/Bq)	(Bq)	(Bq)	(g)	(Bq)	(g)	(Bq)	(g)	(Bq)
²³⁹ Pu	Plutonium	1.000E+09	2.301E+09	15.00	3.452E+10	8.100E-13	1.235E+13	Infinite	15.0	3.45E+10	15.0	3.45E+10	15.0	3.45E+1
²⁴¹ Pu	Plutonium	6.000E+10	3.814E+12	7.50	2.861E+13	3.240E-15	3.086E+15	Mass limit	1.5	5.72E+12	1.5	5.72E+12	1.5	5.72E+1
²³³ U	Uranium	6.000E+09	3.584E+08	15.00	5.377E+09	7.560E-13	1.323E+13	Infinite	15.0	5.38E+09	15.0	5.38E+09	15.0	5.38E+0
²³⁵ U	Uranium	U	8.001E+04	15.00	1.200E+06	7.020E-13	1.425E+13	Infinite	15.0	1.20E+07	15.0	1.20E+06	15.0	1.20E+0
²³⁹ Pu/Be	Pu/Be [3]	1.000E+09	2.301E+09	15.00	3.452E+10	8.100E-13	1.235E+13	9.730E+10	15.0	3.45E+10	15.0	3.45E+10	15.0	3.45E+1
Non-fissile Nuclide	s													
²⁵² Cf	Californium [3]	3.000E+09	1.994E+13		NA	9.510E-13	1.052E+13	4.980E+10	1.00E-02	1.99E+11	2.5E-03	4.98E+10	2.5E-03	4.98E+1
²³⁸ Pu	Plutonium	1.000E+09	6.339E+11		NA	8.910E-13	1.122E+13	Infinite	75.0	4.75E+13	17.7	1.12E+13	4.7	3.00E+1
²⁴⁰ Pu	Plutonium	1.000E+09	8.435E+09		NA	8.370E-13	1.195E+13	Infinite	75.0	6.33E+11	75.0	6.33E+11	75.0	6.33E+1
²⁴² Pu	Plutonium	1.000E+09	1.453E+08		NA	7.830E-13	1.277E+13	Infinite	75.0	1.09E+10	75.0	1.09E+10	75.0	1.09E+10
²³² U	Uranium	1.000E+09	7.923E+11		NA	8.370E-13	1.195E+13	4.440E+08	1.50E-03	1.19E+09	5.60E-04	4.44E+08	5.60E-04	4.44E+0
²³⁴ U	Uranium	6.000E+09	2.313E+08		NA	7.560E-13	1.323E+13	Infinite	1.50E+02	3.47E+10	1.50E+02	3.47E+10	1.50E+02	3.47E+1
²³⁶ U	Uranium	6.000E+09	2.395E+06		NA	6.750E-13	1.481E+13	Infinite	1.50E+02	3.59E+08	1.50E+02	3.59E+08	1.50E+02	3.59E+0
²³⁸ U	Uranium	U	1.245E+04		NA	6.750E-13	1.481E+13	Infinite	1.00E+04	1.24E+08	1.00E+04	1.24E+08	1.00E+04	1.24E+0
²⁴¹ Am	Americium	1.000E+09	1.271E+11	88.00	NA	8.910E-13	1.122E+13	2.770E+13	8.80E+01	1.12E+13	8.80E+01	1.12E+13	2.36E+01	3.00E+1
242 -	Americium [4] (in													
²⁴³ Am	10mm lead pot)	1.000E+09	7.382E+09		NA	1.517E-11	6.592E+11	1.218E+12	8.80E+01	6.50E+11	8.80E+01	6.50E+11	8.80E+01	6.50E+1
²⁴¹ Am/Be	Americium / Beryllium [3]	1 000E+09	1.271E+11	88.00	NA	8.910E-13	1 122F+13	9.730E+10	8 80F+01	1 12F+13	7.66E-01	9.73E+10	7.66E-01	9.73E+1
7411/00	Americium / Boron	1.000L · 03	1.2712-11	00.00	14/1	0.510L 15	1.1222.10	3.100L · 10	0.00L*01	1.12L-10	7.00L 01	3.70L · 10	7.00L 01	3.70L · 1
²⁴¹ Am/B	[3]	1.000E+09	1.271E+11	88.00	NA	8.910E-13	1.122E+13	9.730E+10	8.80E+01	1.12E+13	7.66E-01	9.73E+10	7.66E-01	9.73E+1
	Americium / Lithium													
²⁴¹ Am/Li	[3]	1.000E+09	1.271E+11	88.00	NA	8.910E-13	1.122E+13	9.730E+10	8.00E+00	1.02E+12	7.66E-01	9.73E+10	7.66E-01	9.73E+1
²⁴¹ Am/F	Americium /	4 0005 . 00	4.0745.44	00.00	NIA.	0.0405.40	4.4005.40	0.7005 .40	0.005.04	4.405.40	7.005.04	0.705.40	7.005.04	0.705.4
-··Am/F	Fluorine [3] Americium /	1.000E+09	1.271E+11	88.00	NA	8.910E-13	1.122E+13	9.730E+10	8.80E+01	1.12E+13	7.66E-01	9.73E+10	7.66E-01	9.73E+1
²⁴¹ Am/ ²⁴³ Am/Be	Beryllium [3]	1 000F+09	1.271E+11		NA	8.910E-13	1 122F+13	9.730E+10	8 80F+01	1 12F+13	7.66E-01	9.73E+10	7.66E-01	9.73E+1
7.11.2	Americium / Boron	1.0002 00				0.0102 10	111222 10	0.11002 10	0.002 01		7.002 01	0.1102 10	7.552 51	0.102
²⁴¹ Am/ ²⁴³ Am/B	[3]	1.000E+09	1.271E+11		NA	8.910E-13	1.122E+13	9.730E+10	8.80E+01	1.12E+13	7.66E-01	9.73E+10	7.66E-01	9.73E+10
²⁴¹ Am/ ²⁴³ Am/Li	Americium / Lithium	4 0005 . 00	4.0745.44			0.0405.40	4 4005 : 40	0.7005 : 40	0.005.04	4.405 . 40	7.005.04	0.705.40	7.005.04	0.705.4
Am/E Am/Li	[3] Americium /	1.000E+09	1.271E+11		NA	8.910E-13	1.122E+13	9.730E+10	8.80E+01	1.12E+13	7.66E-01	9.73E+10	7.66E-01	9.73E+10
²⁴¹ Am/ ²⁴³ Am/F	Fluorine [3]	1.000E+09	1.271E+11		NA	8.910E-13	1.122E+13	9.730E+10	8.80E+01	1.12E+13	7.66E-01	9.73E+10	7.66E-01	9.73E+1
²³⁷ Np	Neptunium		2.610E+07	300.00	NA NA	1.020E-12		4.250E+09					1.63E+02	4.25E+0
90Sr	Strontium		5.051E+12	555.00	NA NA	1.810E-13	5.525E+13		1.11E+12				1.09E+01	5.52E+1
²³² Th	Thorium	11	4.061E+03		NA NA	6.430E-13	1.555E+13		1.00E+04				1.00E+04	4.06E+0
²⁴⁴ Cm	Curium		2.996E+12		NA NA	9.300E-13	1.075E+13				3.59E+00		2.00E+00	6.00E+1
Oill	Gurium	2.000L · 09	2.330L · 1Z		IVA	3.500L*15	1.0752113	mining	1.002.04	0.00L · 10	J.JJL . 00	1.00L · 13	2.00L · 00	0.00L 11

^[1] Croft Associates Ltd CTR 83/3, Heat generation by the decay of radioactive isotopes, Issue F dated 16/6/1988 [2] IAEA TS-G-1.1, Advisory Material for the IAEA Regulations for the Safe Transport of Radioactive Material, para 240 [3] Croft Associates Ltd, CS 2000/18, Package 2799E - Dose rates due to neutron emitters, Issue C [4] Croft Associates Ltd, CS 2000/19, Package 2799E - Dose rate and heat output from Am-243, Issue B





Pipeline and Hazardous Materials Safety Administration

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